The World's Reactors No. 57 FFTF OWNER United States Atomic Energy Commission Hanford Engineering Development Laboratory under AEC contract with Westinghouse Hanford Company ARCHITECT - ENGINEER MAIN CONTRACTOR LOCATION Sodium-cooled fast test reactor **GENERAL PERFORMANCE** 400 MW (th) Total power output $7 \times 10^{15} \text{n/cm}^2 \text{s}$ Heat transport systems

36 in (914 mm) 540 kg (Pu 239 + 241)

93 to 108

PuO₂–UO₂ 316 SS

80 MWd/kg

45 MWd/kg

100 full power days

4.715 in (119.7 mm)

0.230 in (5.84 mm) 0.015 in (0.38 mm)

0.475 in (12.06 mm)

0.053 in (1.34 mm)

17.5×10⁶ lb/h

 $(38.6 \times 10^6 \text{ kg/h})$ 1050°F (566°C)

792°F (422°C)

Vertical, single stage, free surface centrifugal

1100 rev/min

14500 gal/min (12100 lmp. gal/min, 500 ft sodium (152 m)

17.4×10⁶ lb/h

965°F (518°C)

707°F (375°C)

1020 rev/min

2000 H.P. 14500 gal/min

915 l/s)

Vertical, single stage,

(12100 Imp. gal/min,

400 ft sodium (122 m)

4 initial/6 ultimate

2.3 MW(th) initial/ 4.3 MW(th) ultimate

1200°F (649°C)

(316°C/538°C)

(2.9×105 kg/h)

316 SS

1.31 × 105 lb/h (max)

600°F (min)/1000°F (max)

free surface centrifugal

316 SS 304 SS

2500 H.P.

304 SS

REACTOR CORE

Core length (active)

Fuel cycle period Core positions

Closed loops Open test assemblies Primary safety rods Operating control rods

Radial reflectors

Reflector material **FUEL ASSEMBLIES**

Clad material

Clad thickness Fuel pin spacer

Poison material Clad material

Flow rate

Radial reflector positions Peripheral control rods (optional)

Lattice pitch (hexagonal)

Fuel pins per assembly

Target burnup, peak

Target burnup, average

CONTROL ASSEMBLIES

Poison pins per assembly

Poison pin clad thickness

PRIMARY SYSTEM

Hot leg temperature

Cold leg temperature

Cold leg material

Design speed

Design flow rate

SECONDARY SYSTEM

Dump heat exchanger and cold leg

CLOSED LOOP IRRADIATION FACILITIES

Maximum test temperature difference 400°F (222°C)

Guard vessels for the reactor vessel, primary pumps and

Inerted galleries for in-containment heat transport systems

Decay heat removal for reactor and closed loops provided by

are required for adequate core cooling following full power

natural circulation (only two of the primary heat transport loops

Zero release design. Gaseous fission product and tritium releases

Hot leg temperature

Cold leg temperature Hot leg material

Pumps

Design speed

perloop

operation).

Design flow rate

Number of closed loops Primary and secondary coolant Total power dissipation capability

Maximum test outlet temperature

Primary and secondary material

intermediate heat exchangers

UNIQUE SAFETY FEATURES

from the plant are below detectable limits

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Nuclear Engineering International,

Test length available Test cross section

Test inlet temperature

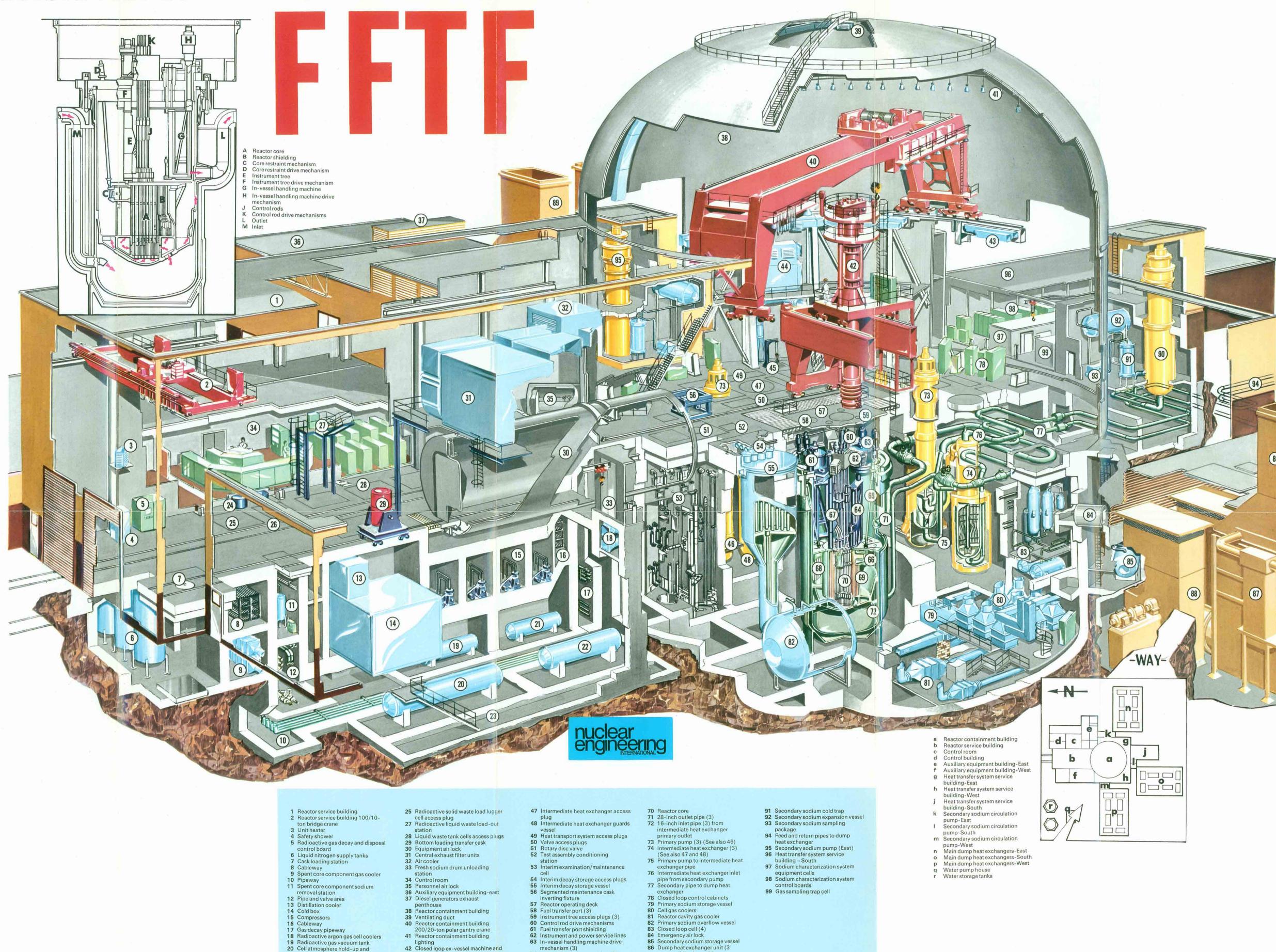
Primary loop flow rate

Type

Poison pin outside diameter

Fuel pin outside diameter

Critical mass **Enrichment zones**



mechanism (3)

67 Instrument tree (3)

69 Radial shield

storage tank

tanks

21 Radioactive gas surge tank

23 Access and operating area

24 Radioactive liquid waste flush

22 Argon processing surge delay tank

transporter

43 Ventilating ducts 44 Air cooler unit

46 Primary pump guard vessel (3)

45 5-ton jib crane

64 In-vessel handling machine (3)

65 Core restraint drive mechanism

66 Core restraint mechanism

68 In-vessel storage module (3)

groups of 4 units each)

88 Fan assembly and air intake

89 Dump heat exchanger (East)

90 Secondary pump (South)

87 Dump heat exchanger heating

FAST FLUX TEST FACILITY