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LOS ALAMOS SCIENTIFIC LABORATORY
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Evaluated Nuclear Data for
Hydrogen in the ENDF/B-II Format

LOS ALAMOS NATIONAL LABORATORY



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**Evaluated Nuclear Data for
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by

L. Stewart
R. J. LaBauve
P. G. Young



EVALUATED NUCLEAR DATA FOR HYDROGEN IN THE ENDF/B-II FORMAT

by

L. Stewart, R. J. LaBauve, and P. G. Young

ABSTRACT

The following nuclear data are given for hydrogen in the energy range from 1.0×10^{-5} eV to 20.0 MeV.

- File 1. The general information file includes a brief description of the data to follow.
- File 2. Values for nuclear spin and effective scattering radius are given in the resonance file.
- File 3. Smooth cross-section data are given for the total cross section, the free-atom elastic scattering cross section, and the radiative capture cross section; data for $\bar{\mu}$, ξ , and γ are also included.
- File 4. The angular distributions for elastic scattering are given as probability vs cosine of the scattering angle.
- File 7. The free-atom-scattering cross section is the only information provided at thermal.
- File 12. Secondary gamma-ray production multiplicities for capture, which are equal to one, are given in this file.
- File 14. Gamma-ray angular distributions are provided for the single radiative capture gamma ray.

INTRODUCTION

This evaluation for hydrogen (MAT = 1148) differs from the previous ENDF/B evaluation (MAT = 1001) in that the elastic scattering data were taken from recent work by Hopkins and Breit¹ and the data for radiative capture were taken from recent work by Horsley.² Also, gamma-ray production data, not given in the MAT = 1001 evaluation, are included. A complete listing for MAT = 1148 is given in the Appendix.

FILE 1: GENERAL INFORMATION

A brief summary of the data to follow is given in File 1. The atomic mass for hydrogen was taken to be 1.007825 from the May 1969 "Chart of the Nuclides."³

FILE 2: RESONANCE INFORMATION

Nuclear spin and effective scattering radius are given in this file. An effective scattering

radius of 1.2756×10^{-12} cm is consistent with a potential scattering cross section of 20.449 b, as determined from $4\pi a^2$. Singlet and triplet scattering radii are not included.

FILE 3: SMOOTH CROSS SECTIONS

Total cross sections (MT = 1) were obtained by adding the elastic scattering and radiative capture cross sections at all energies (1.0×10^{-5} eV to 20.0 MeV). The hydrogen total cross sections are shown in Fig. 1.

The elastic scattering cross sections (MT = 2) were taken from an extensive theoretical treatment of fast neutron measurements by Hopkins and Breit.¹ In this work, a consistent set of cross sections and angular distributions were obtained by using a set of phase shifts previously determined at Yale University.⁴ Tabular values of the elastic scattering cross section are given in Ref. 1 for only a

few energies, the two lowest points being 100 and 200 keV. The phase shift program and the Yale phase shifts were provided by Hopkins¹ so that many intermediate points could be calculated. At 0.1 keV, the lowest energy recommended for running this program, the scattering cross section is 20.4488 b. This value is in excellent agreement with the thermal cross section (20.442 ± 0.023 b) derived by Davis and Barschall⁵ from a revised value of the effective range obtained by determining the best values of the neutron energies from many experiments below 5 MeV performed since 1950. Therefore, for this evaluation, the free-atom-scattering cross section is assumed to be constant below 100 eV and equal to the value calculated from the Yale phase shifts at 100 eV, giving a thermal cross section of 20.449 b. At higher energies, these theoretical predictions are in excellent agreement with the recent measurements of Davis⁶ giving an average value of 0.84 for the square of the deviation for energies below 20.0 MeV. The elastic cross section for hydrogen from 1.0×10^{-5} eV to 20.0 MeV is shown in Fig. 2.

The cross sections for radiative capture (MT = 102) were taken from the 1966 publication of Horsley,² where a value of 332 mb was adopted for the thermal value. Deuteron photodisintegration cross sections were also employed in deriving radiative capture in Horsley's report. Although the Nuclear Data article by Horsley² was referenced for MAT = 1001, the values were taken from an early version described in AWRE O-23/65, and these were later revised for the Nuclear Data article. The latter report (Ref. 2) has been used for this evaluation, as suggested by Horsley. The radiative capture cross section for MAT = 1148 from 1.0×10^{-5} eV to 20.0 MeV is shown in Fig. 3.

The average value of the cosine in the laboratory system ($\bar{\mu}_L$) for elastic scattering (MT = 251) was derived from the secondary angular distributions in File 4 (MT = 4). Values for $\bar{\mu}_L$ from 1.0×10^{-5} eV to 20.0 MeV are shown in Fig. 4.

Values for ξ , the average logarithmic energy change per collision (MT = 252), and for γ , the Goertzel-Greuling constant (MT = 253), are taken equal to 1 over the range 1.0×10^{-5} eV to 20.0 MeV, following the MT = 1001 evaluation.

FILE 4: SECONDARY ANGULAR DISTRIBUTIONS

Angular distributions of secondary neutrons resulting from elastic scattering are tabulated from 1.0×10^{-5} eV to 20.0 MeV. Distributions at 0.1, 5, 10, 20, and 30 MeV are provided by Ref. 1; additional and intermediate data were calculated by using the Hopkins-Breit phase shift program and the Yale phase shifts. As shown in Figs. 5 through 16, the angular distributions above 100 keV are neither isotropic below 10 MeV, nor are they symmetric about 90° at higher energies as assumed in the earlier version (MAT = 1001). At 100 keV, the angular distributions are assumed to be isotropic because the 180/0° ratio is very nearly unity (1.0011). At 500 keV, this ratio approaches 1.005; therefore, the pointwise normalized probabilities as a function of the cosine of the scattering angle are provided at 1.0×10^{-5} eV (isotropic), 100 keV (isotropic), 500 keV, and at 1-MeV intervals from 1 to 20 MeV.

FILE 5: THERMAL DATA

Free-atom cross sections specified from 1.0×10^{-5} eV to 5 eV are included in this file.

FILE 12: PHOTON PRODUCTION CROSS SECTIONS

A multiplicity representation is used to describe the single hydrogen radiative capture gamma ray from 1.0×10^{-5} eV to 20.0 MeV. The multiplicity is referred to MT = 102 in File 3 and is unity at all neutron energies. To adequately represent the gamma-ray energy for MeV-incident neutrons, the neutron energy region from 0.2 to 20 MeV is divided into 16 different energy bands, and the gamma-ray energy is tabulated for each neutron energy band as

$$\bar{E}_\gamma = 2.225 \times 10^6 + \bar{E}_n / 2 \quad (\text{eV}),$$

where \bar{E}_n is the neutron energy at the midpoint of the band in eV. The value 2.225×10^6 eV corresponds to the deuteron binding energy; that is, the small energy change due to the nuclear recoil that accompanies gamma emission has been ignored.

FILE 14: GAMMA-RAY ANGULAR DISTRIBUTIONS

The gamma-ray angular distributions are assumed to be isotropic at all neutron energies from 1.0×10^{-5} eV to 20.0 MeV.

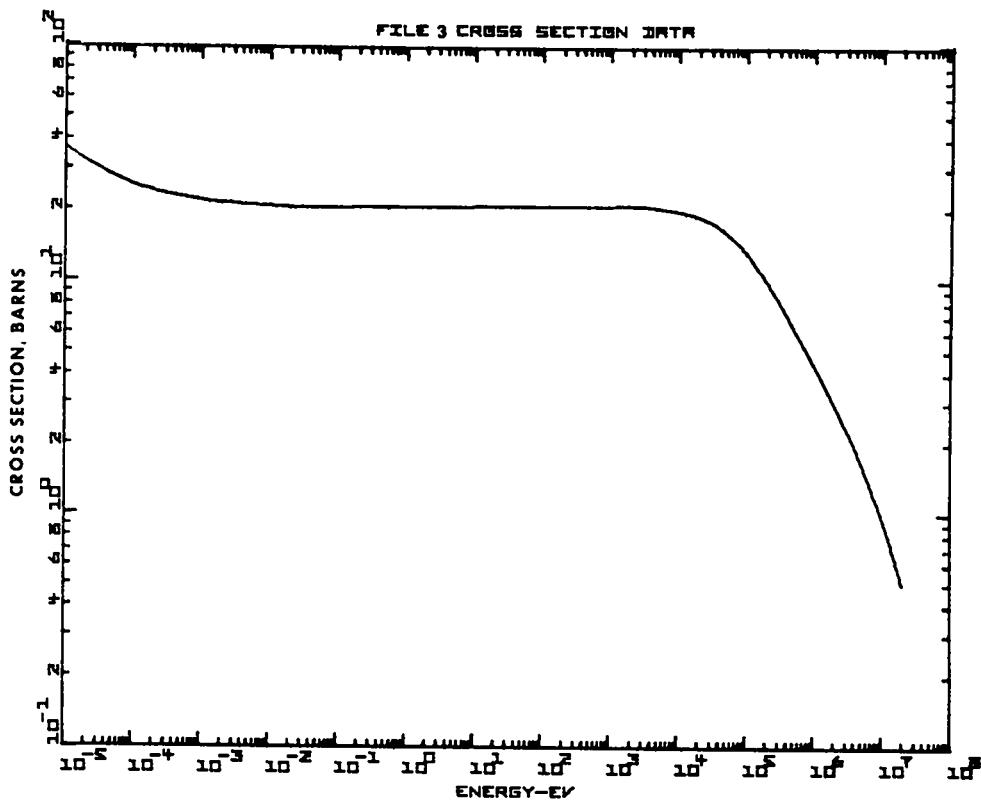


Fig. 1. Total cross section (MT = 1).

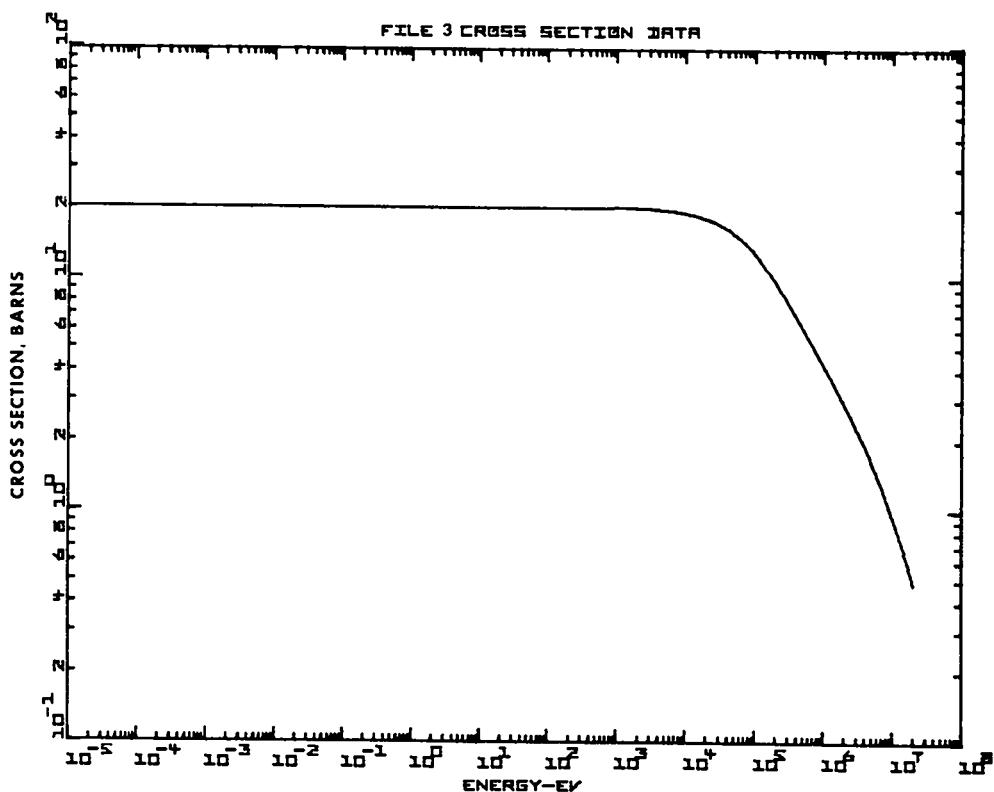


Fig. 2. Elastic scattering cross section (MT = 2).

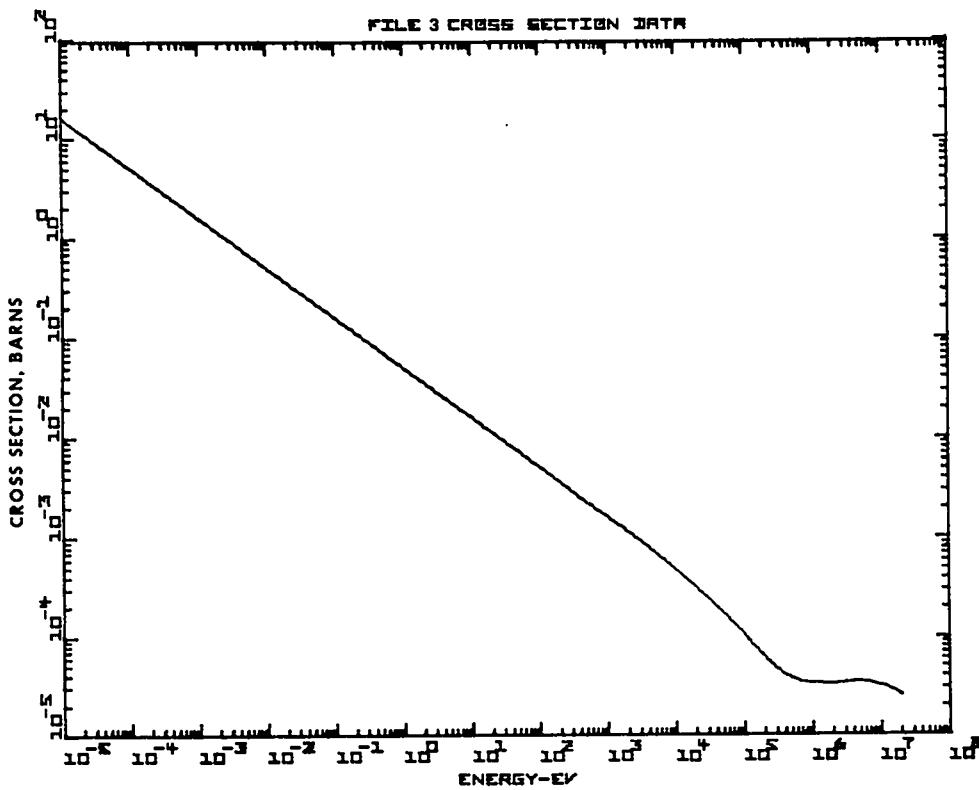


Fig. 3. Radiative capture cross section (MT = 102).

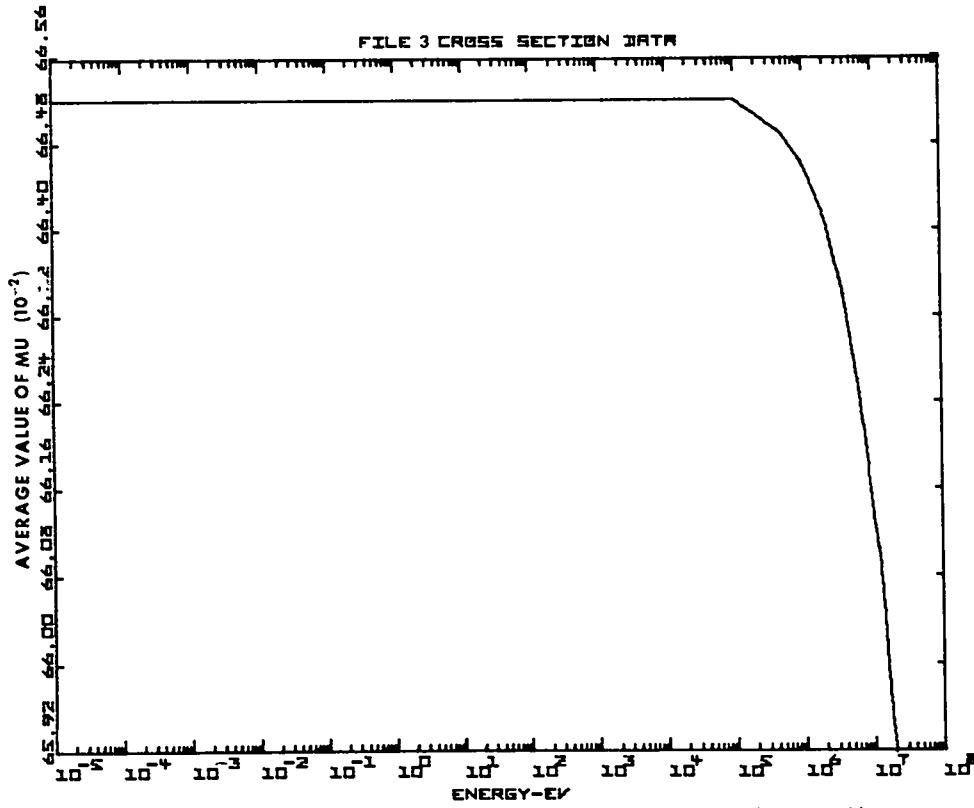


Fig. 4. Average value of cosine in laboratory system (MT = 251).

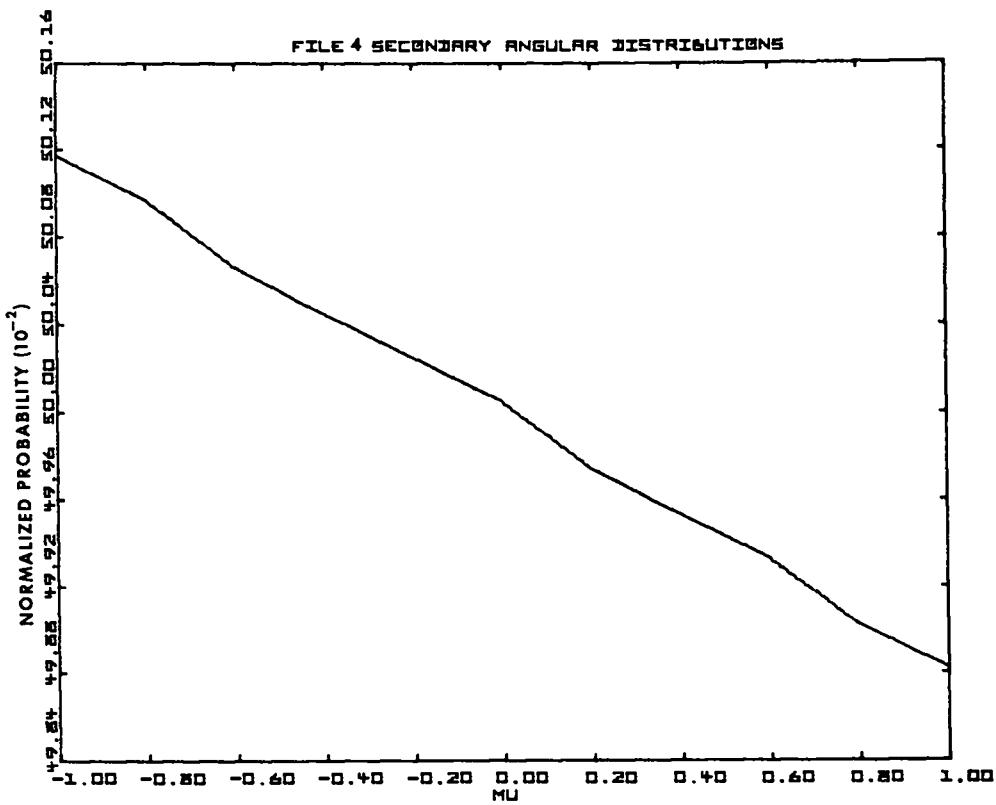


Fig. 5. Angular distribution for 0.5 MeV.

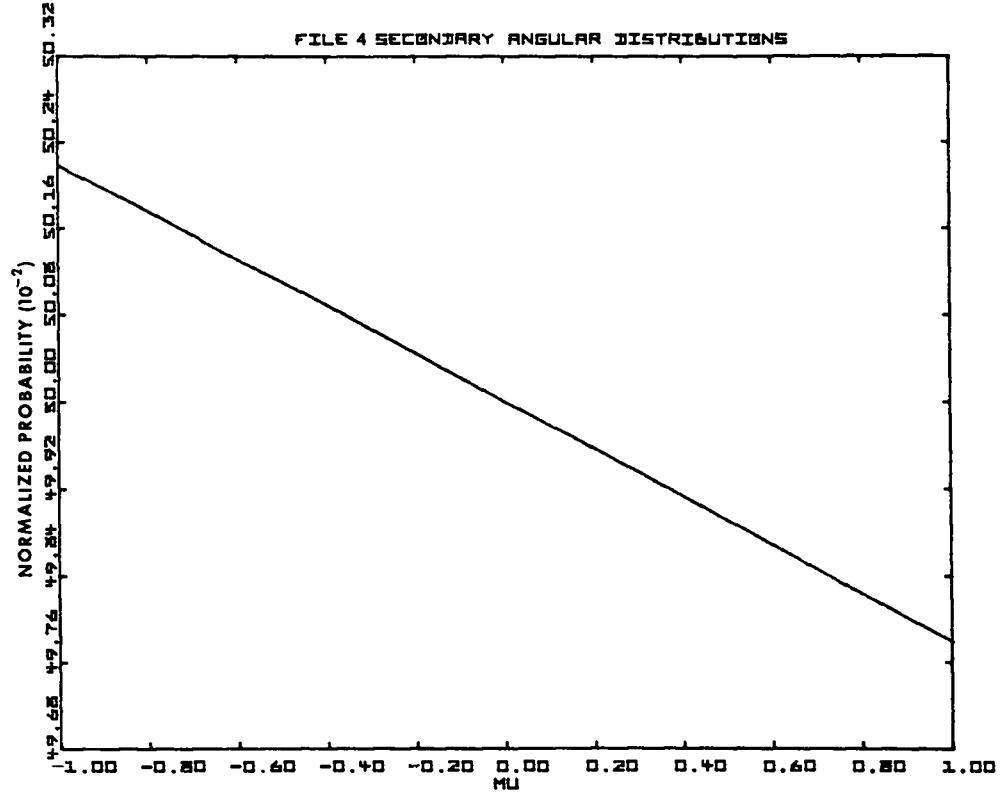


Fig. 6. Angular distribution for 1.0 MeV.

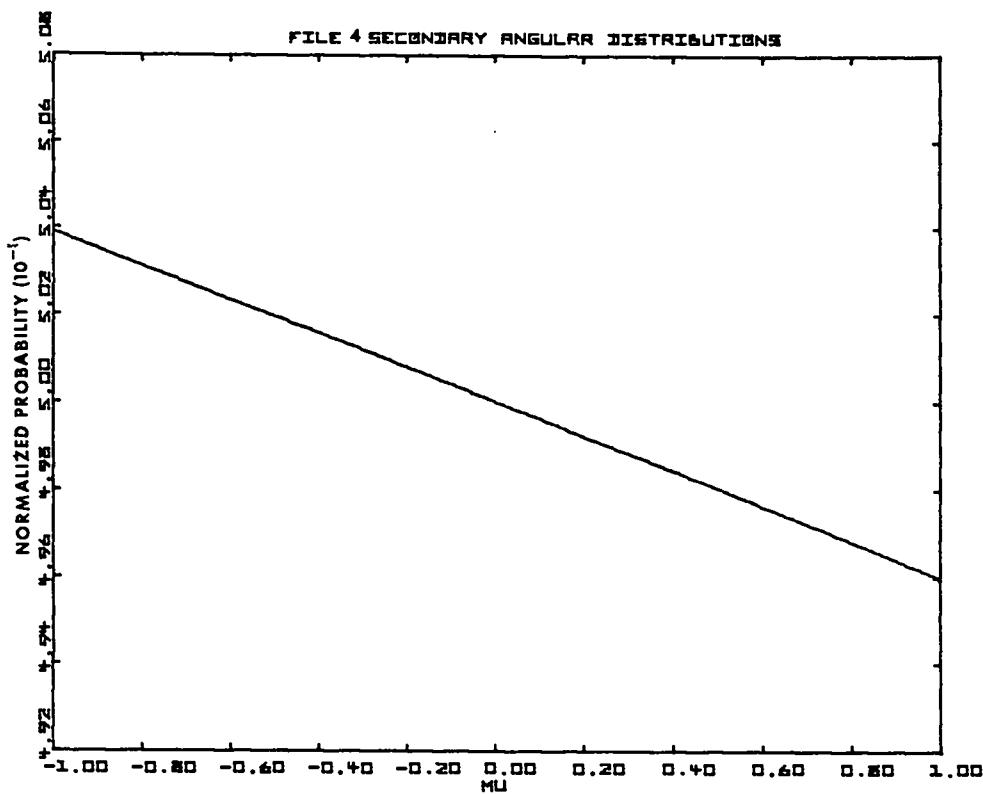


Fig. 7. Angular distribution for 2.0 MeV.

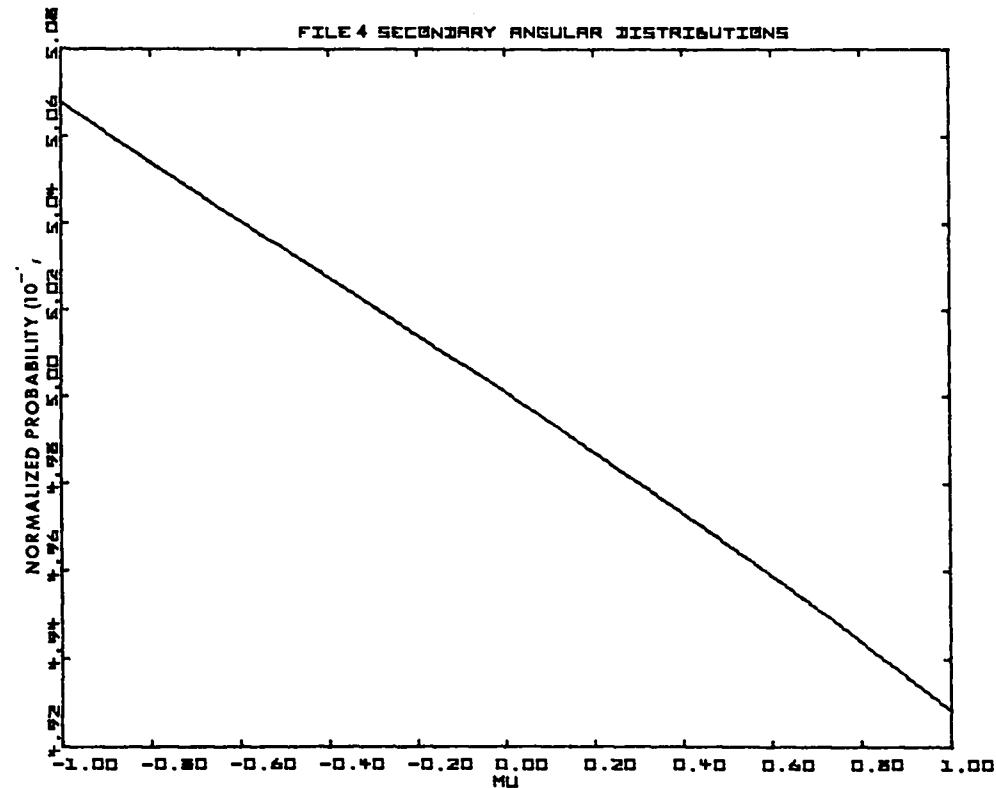


Fig. 8. Angular distribution for 4.0 MeV.

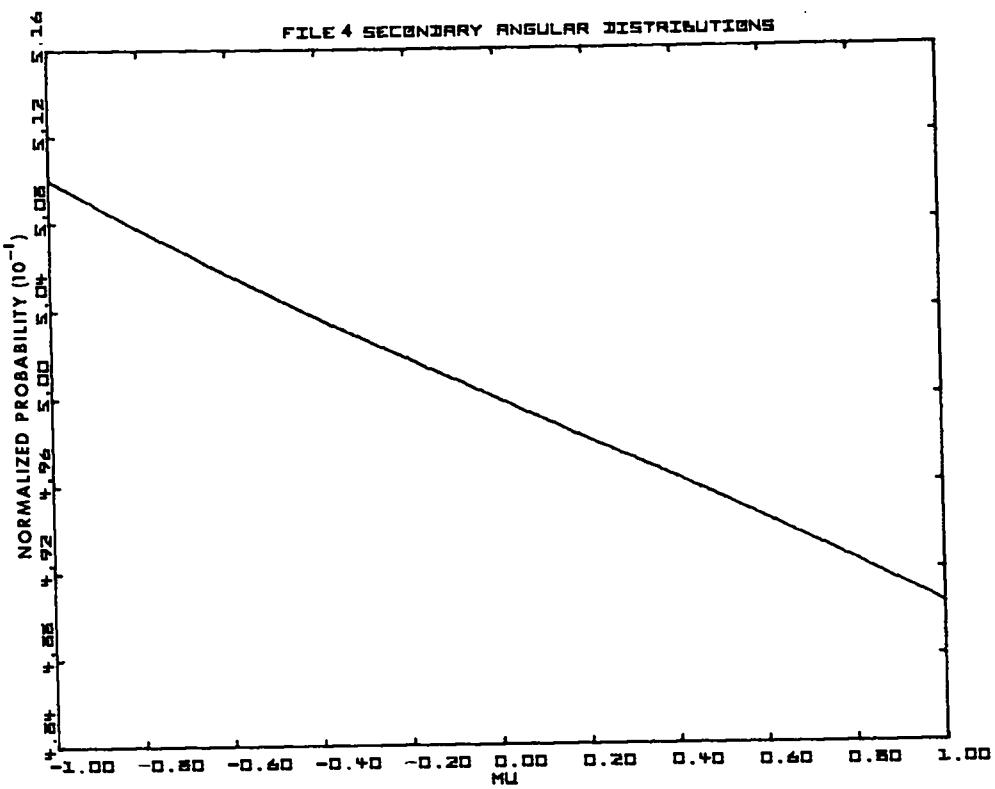


Fig. 9. Angular distribution for 6.0 MeV.

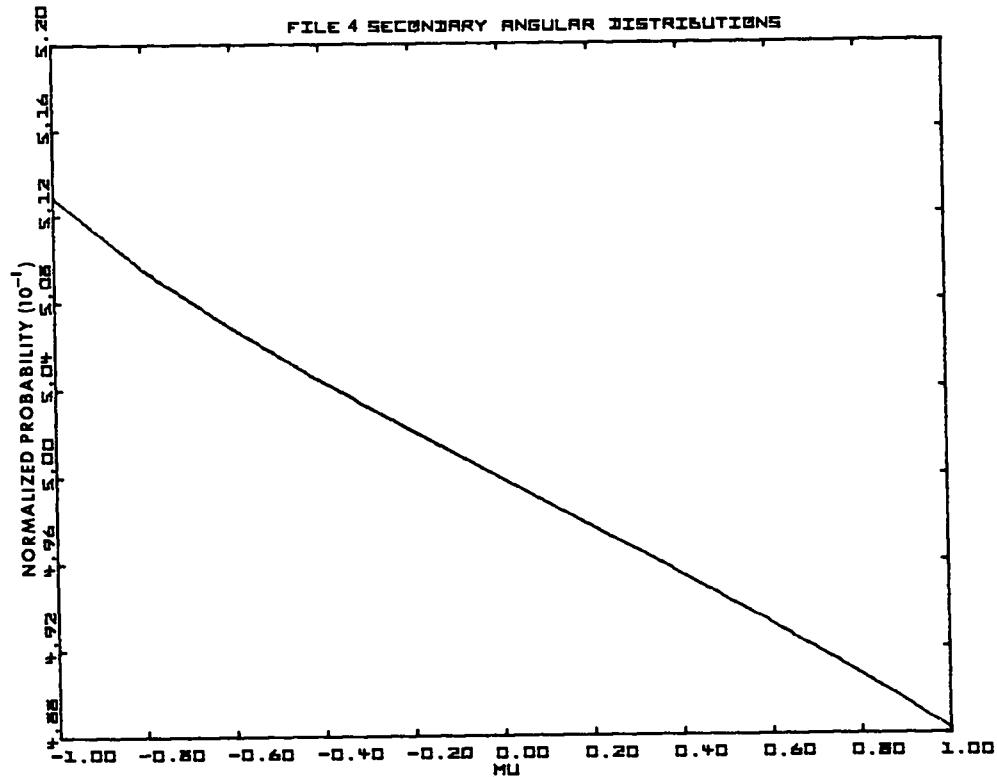


Fig. 10. Angular distribution for 8.0 MeV.

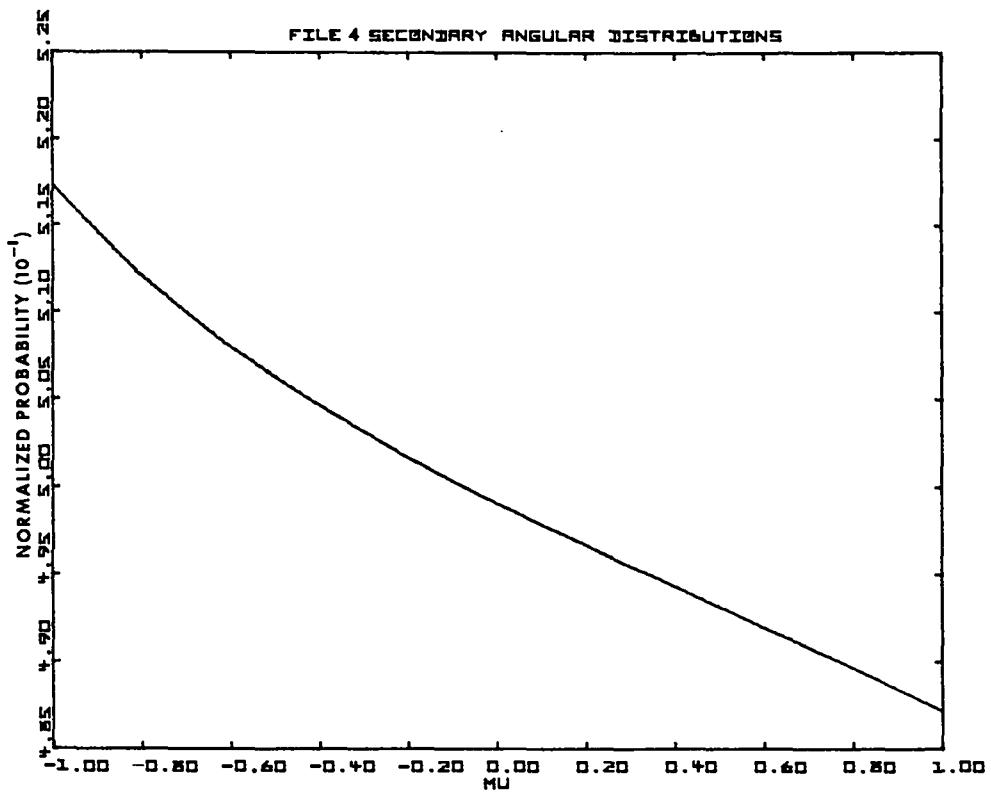


Fig. 11. Angular distribution for 10.0 MeV.

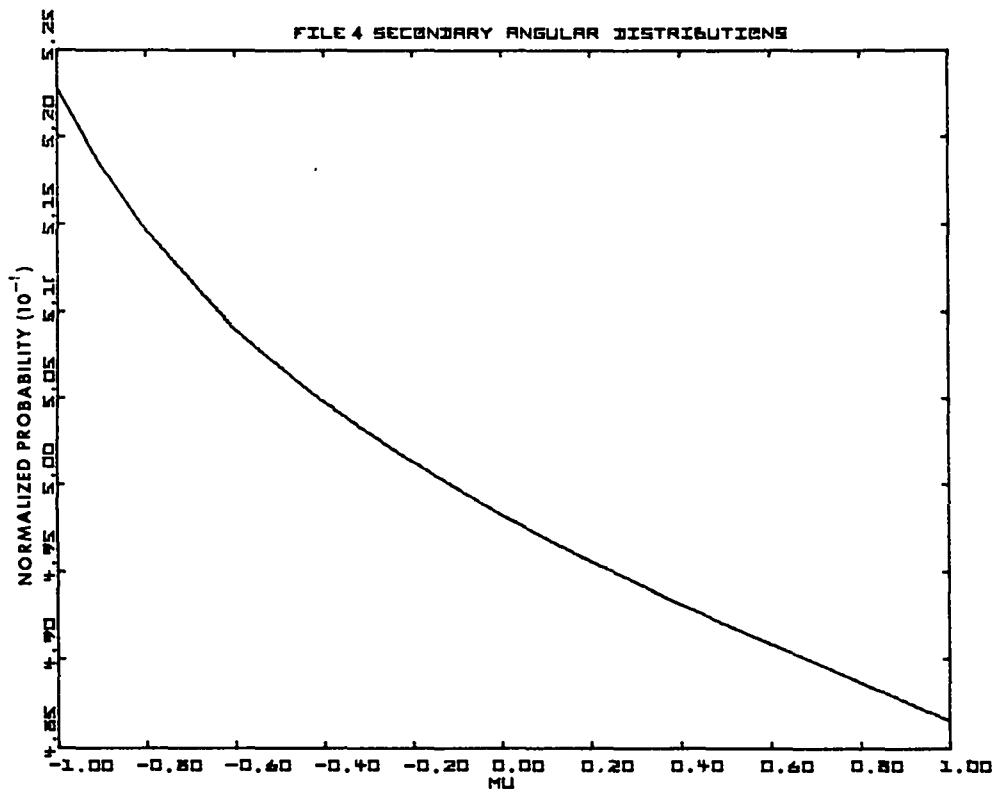


Fig. 12. Angular distribution for 12.0 MeV.

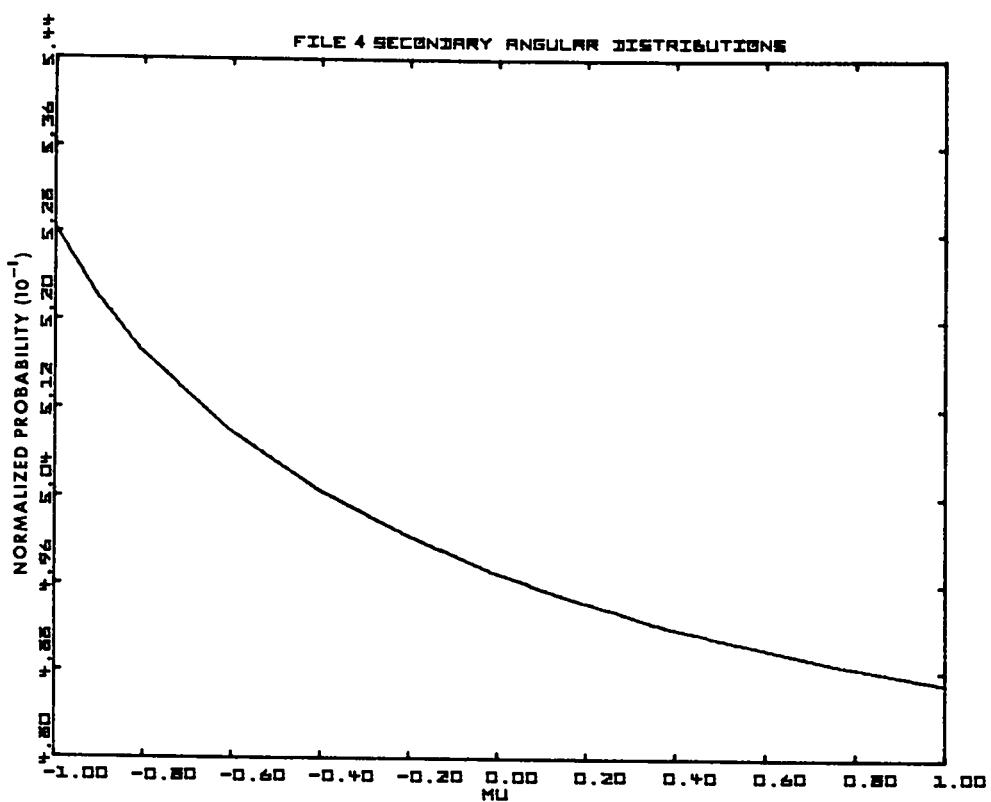


Fig. 13. Angular distribution for 14.0 MeV.

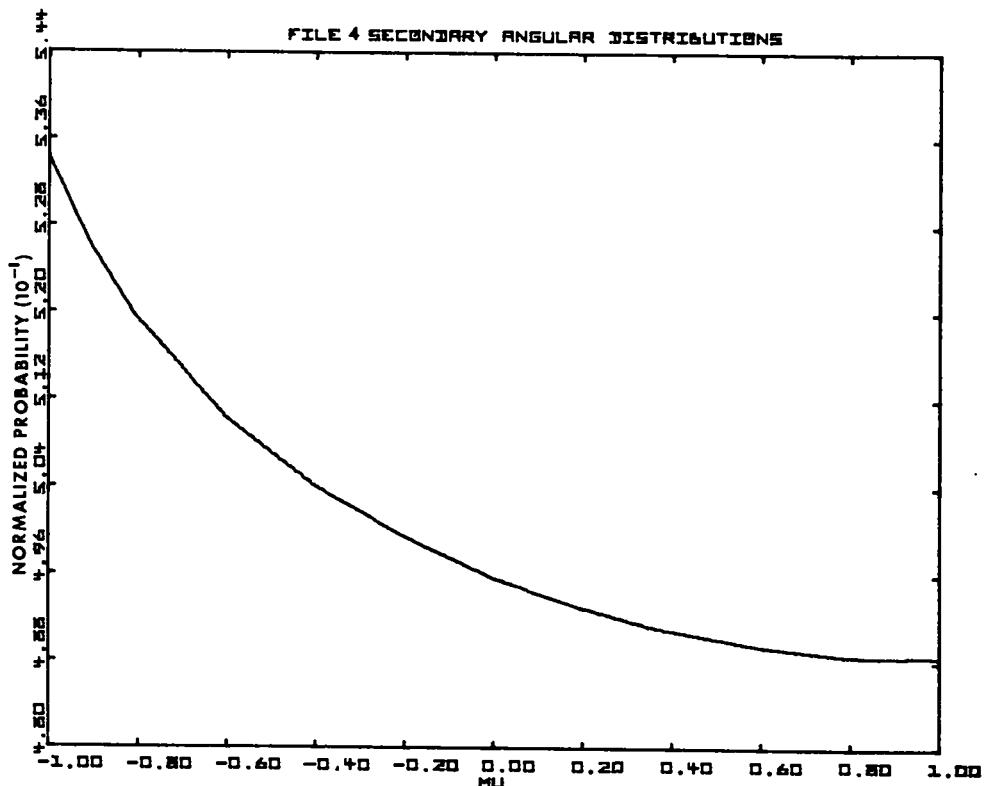


Fig. 14. Angular distribution for 16.0 MeV.

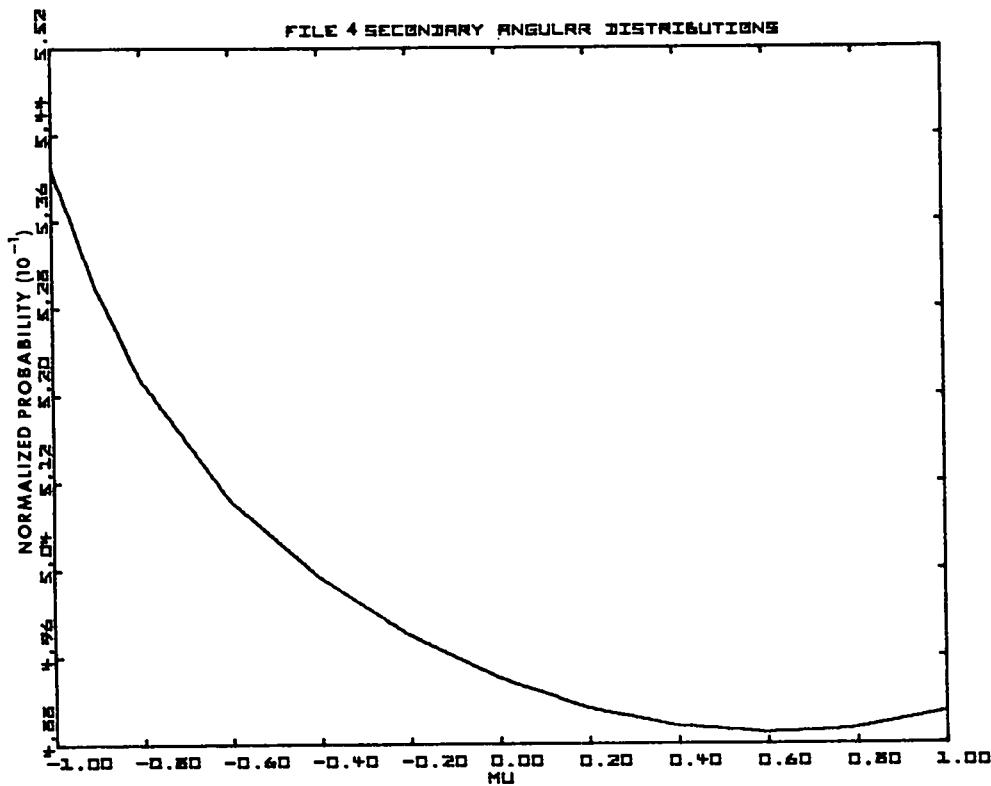


Fig. 15. Angular distribution for 18.0 MeV.

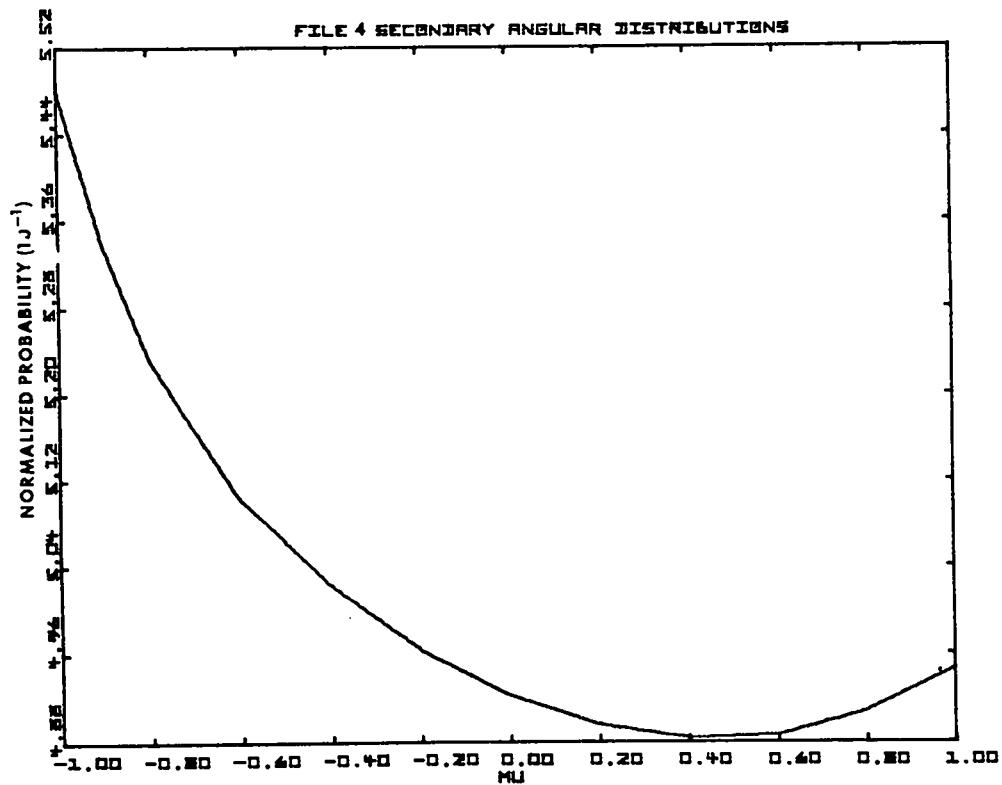


Fig. 16. Angular distribution for 20.0 MeV.

REFERENCES

1. J. C. Hopkins and G. Breit, "The H(n,n)H Scattering Observables Required for High Precision Fast-Neutron Measurements," submitted to Nuclear Data for publication; private communication prior to publication (1970).
2. A. Horsley, "Neutron Cross Sections of Hydrogen in the Energy Range 0.001 eV - 20 MeV," Nuclear Data A 2, 243 (1966) and private communication (1970).
3. "Chart of the Nuclides," Battelle Memorial Institute, May 1969.
4. R. E. Seamon, K. A. Friedman, G. Breit, R. D. Haracz, J. M. Holt, and A. Prakash, Phys. Rev. 165, 1579 (1968).
5. J. C. Davis and H. H. Barschall, "Adjustments in the n-p Singlet Effective Range," Phys. Letters 27B, 636 (1968).
6. J. C. Davis, BAPS 11 15, 474 (1970) and private communication (1970).

APPENDIX

LISTING OF HYDROGEN EVALUATION

HYDROGEN IN ENDF/B-II FORMAT. MAT = 1148					
1.001 E+03	9.9917E-01	0	0	1148-0 -0	0
0.0	0.0	0	0	121148 1451	1
			67	01148 1451	2
				1148 1451	3
				1148 1451	4
				1148 1451	5
				1148 1451	6
				1148 1451	7
				1148 1451	8
				1148 1451	9
				1148 1451	10
				1148 1451	11
				1148 1451	12
				1148 1451	13
				1148 1451	14
				1148 1451	15
				1148 1451	16
				1148 1451	17
				1148 1451	18
				1148 1451	19
				1148 1451	20
				1148 1451	21
				1148 1451	22
				1148 1451	23
				1148 1451	24
				1148 1451	25
				1148 1451	26
				1148 1451	27
				1148 1451	28
				1148 1451	29
				1148 1451	30
				1148 1451	31
				1148 1451	32
				1148 1451	33
				1148 1451	34
				1148 1451	35
				1148 1451	36
				1148 1451	37
				1148 1451	38
				1148 1451	39
				1148 1451	40
				1148 1451	41
				1148 1451	42
				1148 1451	43
				1148 1451	44
				1148 1451	45
				1148 1451	46
				1148 1451	47
				1148 1451	48
				1148 1451	49

HYDROGEN FREE ATOM CROSS SECTIONS
ENTRY BY L. STEWART, R.J. LABAUVE, AND P.G. YOUNG
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LOS ALAMOS, NEW MEXICO 87544
OCTOBER 20, 1970

MF=1

MT=451 • ATOMIC MASS=1.007825

MF=2

MT=151 • SCATTERING LENGTH=1.2756E-12 CM.

MF=3

MT= 1 • TOTAL CROSS SECTIONS --- THE TOTAL CROSS SECTIONS ARE OBTAINED BY ADDING THE PLASTIC SCATTERING AND RADIATIVE CAPTURE CROSS SECTIONS AT ALL ENERGIES. 1.0E-05 EV TO 20 MEV.

MT= 2 • ELASTIC SCATTERING --- FROM AN EXTENSIVE THEORETICAL TREATMENT OF FAST NEUTRON MEASUREMENTS BY J. C. HOPKINS(LASL) AND G. BREIT(STATE UNIVERSITY OF NEW YORK). 1.0E-05 FV TO 20 MEV.

MT=102, RADIATIVE CAPTURE --- THESE CROSS SECTIONS ARE TAKEN FROM THE 1966 PUBLICATION OF A. HORSLEY WHERE A VALUE OF 332 MR WAS ADOPTED FOR THE THERMAL VALUE. 1.0E-05 EV TO 20 MEV.

MT=251, AVERAGE VALUE OF COSINE OF SCATTERING ANGLE IN LAB SYSTEM. 1.0E-05 EV TO 20 MEV.

MT=252, AVERAGE LOGARITHMIC ENERGY CHANGE PER COLLISION, TAKEN AS 1, FROM 1.0E-05 EV TO 20 MEV.

MT=253, GAMMA, TAKEN AS 1, FROM 1.0E-05 EV TO 20 MEV.

MF=4

MT= 2, NEUTRON ELASTIC SCATTERING ANGULAR DISTRIBUTIONS IN THE CENTER OF MASS SYSTEM--GIVEN AS NORMALIZED POINTWISE PROBABILITIES.

MF=7		1148 1451 50
		1148 1451 51
		1148 1451 52
		1148 1451 53
MT= 4, .00001 TO 5 EV FREE GAS SIGMA=20.449 BARNs.		1148 1451 54
		1148 1451 55
MF=12		1148 1451 56
		1148 1451 57
MT=102, GAMMA RAY MULTIPLICITIES --- MULTIPLICITY, (REFERRED TO MT=102, MF=3), IS UNITY AT ALL NEUTRON ENERGIES. SIXTEEN ENERGY BANDS ARE GIVEN FROM .2 MEV TO 20 MEV, AND THE AVERAGE GAMMA RAY ENERGY, EAG, IS DETERMINED FROM THE AVERAGE NEUTRON ENERGY, EAN, IN THE BAND BY EAG=2.225E+06+EAN/2., RECOIL ENERGY IGNORED.		1148 1451 58
		1148 1451 59
		1148 1451 60
		1148 1451 61
		1148 1451 62
		1148 1451 63
		1148 1451 64
MF=14		1148 1451 65
		1148 1451 66
MT=102, GAMMA RAY ANGULAR DISTRIBUTION --- ASSUMED ISOTROPIC		1148 1451 67

HYDROGEN IN ENDF/B-II FORMAT. MAT = 1148	1148-0 -0 0
1.001 E+03 9.9917E-01 0 0 0	121148 1451 1
0.0 0.0 0 0 67	01148 1451 2
	1148 1451 3
HYDROGEN FREE ATOM CROSS SECTIONS	1148 1451 4
ENTRY BY L. STEWART, R.J. LABAUVE, AND P.G. YOUNG	1148 1451 5
LOS ALAMOS SCIENTIFIC LABORATORY	1148 1451 6
LOS ALAMOS, NEW MEXICO 87544	1148 1451 7
OCTOBER 20, 1970	1148 1451 8
	1148 1451 9
MF=1	1148 1451 10
MT=451, ATOMIC MASS=1.007825	1148 1451 11
	1148 1451 12
MF=2	1148 1451 13
MT=151, SCATTERING LENGTH=1.2756E-12 CM.	1148 1451 14
	1148 1451 15
MF=3	1148 1451 16
MT= 1, TOTAL CROSS SECTIONS --- THE TOTAL CROSS SECTIONS ARE	1148 1451 17
OBTAINED BY ADDING THE ELASTIC SCATTERING AND	1148 1451 18
RADIATIVE CAPTURE CROSS SECTIONS AT ALL ENERGIES.	1148 1451 19
1.0E-05 EV TO 20 MEV.	1148 1451 20
	1148 1451 21
	1148 1451 22
	1148 1451 23
	1148 1451 24
MT= 2, ELASTIC SCATTERING --- FROM AN EXTENSIVE THEORETICAL TREATMENT OF FAST NEUTRON MEASUREMENTS	1148 1451 25
BY J. C. HOPKINS(LASL) AND G. BREIT(STATE	1148 1451 26
UNIVERSITY OF NEW YORK).	1148 1451 27
1.0E-05 EV TO 20 MEV.	1148 1451 28
	1148 1451 29
	1148 1451 30
MT=102, RADIATIVE CAPTURE --- THESE CROSS SECTIONS ARE TAKEN	1148 1451 31
FROM THE 1966 PUBLICATION OF A. HORSLEY WHERE A VALUE	1148 1451 32
OF 332 MB WAS ADOPTED FOR THE THERMAL VALUE.	1148 1451 33
1.0E-05 EV TO 20 MEV.	1148 1451 34
	1148 1451 35
MT=251, AVERAGE VALUE OF COSINE OF SCATTERING ANGLE	1148 1451 36
IN LAB SYSTEM.	1148 1451 37
1.0E-05 EV TO 20 MEV.	1148 1451 38
	1148 1451 39
MT=252, AVERAGE LOGARITHMIC ENERGY CHANGE PER COLLISION, TAKEN	1148 1451 40
AS 1, FROM 1.0E-05 EV TO 20 MEV.	1148 1451 41
	1148 1451 42
MT=253, GAMMA, TAKEN AS 1, FROM 1.0E-05 EV TO 20 MEV.	1148 1451 43
	1148 1451 44
MF=4	1148 1451 45
MT= 2, NEUTRON ELASTIC SCATTERING ANGULAR DISTRIBUTIONS IN	1148 1451 46
THE CENTER OF MASS SYSTEM--GIVEN AS NORMALIZED	1148 1451 47
POINTWISE PROBABILITIES.	1148 1451 48
	1148 1451 49

MF=7
 MT= 4. .00001 TO 5 FV FREE GAS SIGMA=20.449 BARNS.
 MF=12
 MT=102. GAMMA RAY MULTIPLICITIES --- MULTIPLICITY, (REFERRED
 TO MT=102. MF=3), IS UNITY AT ALL NEUTRON ENERGIES.
 SIXTEEN ENERGY BANDS ARE GIVEN FROM .2 MEV TO 20 MEV.
 AND THE AVERAGE GAMMA RAY ENERGY, EAG, IS DETERMINED
 FROM THE AVERAGE NEUTRON ENERGY, EAN. IN THE BAND BY
 $EAG=2.225E+06+EAN/2.$, RECOIL ENERGY IGNORED.
 MF=14
 MT=102. GAMMA RAY ANGULAR DISTRIBUTION --- ASSUMED ISOTROPIC
 AT ALL NEUTRON ENERGIES.

1	451	81	1148	1451	70					
2	151	4	1148	1451	71					
3	1	48	1148	1451	72					
3	2	48	1148	1451	73					
3	102	48	1148	1451	74					
3	251	8	1148	1451	75					
3	252	4	1148	1451	76					
3	253	4	1148	1451	77					
4	2	88	1148	1451	78					
7	4	4	1148	1451	79					
12	102	70	1148	1451	80					
14	102	1	1148	1451	81					
			1148	1	82					
			1148	0	83					
1.001	F+03	9.9917E-01	0	0	1	0	1148	2151	84	
1.001	E+03	1.0000E+00	0	0	1	0	01148	2151	85	
1.0	F-05	1.0	E+05	0	0	0	01148	2151	86	
5.0	F-01	1.2756E+00	0	0	0	0	01148	2151	87	
0.0	0.0	0	0	0	0	0	01148	2	0	88
0.0	0.0	0	0	0	0	0	01148	0	0	89
1.001	F+03	9.9917E-01	0	0	0	0	01148	3	1	90
0.0	0.0	0	0	0	1	134	1148	3	1	91
134		5				1148	3	1	92	
1.0000F-05	3.7148F+01	2.0000E-05	3.2257E+01	5.0000E-05	2.7917E+01	1148	3	1	93	
1.0000E-04	2.5730E+01	2.0000E-04	2.4183E+01	5.0000F-04	2.2811E+01	1148	3	1	94	
1.0000E-03	2.2119E+01	2.0000E-03	2.1630E+01	5.0000E-03	2.1196E+01	1148	3	1	95	
1.0000E-02	2.0977E+01	2.5300E-02	2.0780E+01	1.0000E+02	2.0450E+01	1148	3	1	96	
1.0000F+03	2.0330E+01	2.0000E+03	2.0200E+01	3.0000E+03	2.0070E+01	1148	3	1	97	
4.0000E+03	1.9940E+01	5.0000E+03	1.9820E+01	6.0000E+03	1.9690E+01	1148	3	1	98	
8.0000F+03	1.9450E+01	1.0000E+04	1.9210E+01	1.5000E+04	1.8650E+01	1148	3	1	99	
2.0000F+04	1.8130E+01	2.5000E+04	1.7630E+01	3.0000F+04	1.7170E+01	1148	3	1	100	
3.5000F+04	1.6740E+01	4.0000E+04	1.6330E+01	4.5000E+04	1.5940E+01	1148	3	1	101	
5.0000F+04	1.5580E+01	5.5000E+04	1.5230E+01	6.0000E+04	1.4900E+01	1148	3	1	102	
6.5000F+04	1.4590E+01	7.0000E+04	1.4290E+01	7.5000F+04	1.4010F+01	1148	3	1	103	
8.0000F+04	1.3740E+01	8.5000E+04	1.3480E+01	9.0000F+04	1.3240F+01	1148	3	1	104	
9.5000F+04	1.3000E+01	1.0000E+05	1.2770E+01	1.1000E+05	1.2350E+01	1148	3	1	105	
1.2000E+05	1.1960E+01	1.3000E+05	1.1610E+01	1.4000E+05	1.1280E+01	1148	3	1	106	
1.5000E+05	1.0970E+01	1.6000E+05	1.0670E+01	1.7000E+05	1.0400E+01	1148	3	1	107	
1.8000E+05	1.0140E+01	1.9000E+05	9.8980E+00	2.0000E+05	9.6710E+00	1148	3	1	108	
2.2000E+05	9.2580E+00	2.4000E+05	8.8920E+00	2.6000E+05	8.5620E+00	1148	3	1	109	
2.8000E+05	8.2620F+00	3.0000E+05	7.9870E+00	3.2000E+05	7.7340E+00	1148	3	1	110	
3.4000E+05	7.5010E+00	3.6000E+05	7.2840E+00	3.8000E+05	7.0830E+00	1148	3	1	111	
4.0000E+05	6.8970E+00	4.2000E+05	6.7250E+00	4.4000E+05	6.5650E+00	1148	3	1	112	
4.6000E+05	6.4150E+00	4.8000E+05	6.2750E+00	5.0000E+05	6.1430E+00	1148	3	1	113	
5.5000E+05	5.8450E+00	6.0000E+05	5.5840E+00	6.5000E+05	5.3540E+00	1148	3	1	114	
7.0000E+05	5.1480E+00	7.5000E+05	4.9640E+00	8.0000E+05	4.7970E+00	1148	3	1	115	
8.5000E+05	4.6450F+00	9.0000E+05	4.5060E+00	9.5000E+05	4.3780E+00	1148	3	1	116	
1.0000E+06	4.2610E+00	1.1000E+06	4.0510E+00	1.2000E+06	3.8680E+00	1148	3	1	117	
1.3000E+06	3.7060E+00	1.4000E+06	3.5610E+00	1.5000E+06	3.4290E+00	1148	3	1	118	
1.6000E+06	3.3090E+00	1.7000E+06	3.1980E+00	1.8000F+06	3.0970E+00	1148	3	1	119	

1.0000E-05	1.6699E+01	2.0000E-05	1.1808E+01	5.0000E-05	7.4682E+00	1148	3102	191
1.0000E-04	5.2808E+00	2.0000E-04	3.7341E+00	5.0000E-04	2.3616E+00	1148	3102	192
1.0000E-03	1.6699E+00	2.0000E-03	1.1808E+00	5.0000E-03	7.4682E-01	1148	3102	193
1.0000E-02	5.2808E-01	2.5300E-02	3.3200E-01	1.0000E+02	5.2770E-03	1148	3102	194
1.0000E+03	1.6590E-03	2.0000E+03	1.1926E-03	3.0000E+03	9.7008E-04	1148	3102	195
4.0000E+03	8.3290E-04	5.0000E+03	7.3747E-04	6.0000E+03	6.6619E-04	1148	3102	196
8.0000E+03	5.6518E-04	1.0000E+04	4.9580E-04	1.5000E+04	3.8782E-04	1148	3102	197
2.0000E+04	3.2386E-04	2.5000E+04	2.8064E-04	3.0000E+04	2.4909E-04	1148	3102	198
3.5000E+04	2.2248E-04	4.0000E+04	2.0553E-04	4.5000E+04	1.8970E-04	1148	3102	199
5.0000E+04	1.7646E-04	5.5000E+04	1.6518E-04	6.0000E+04	1.5544E-04	1148	3102	200
6.5000E+04	1.4693E-04	7.0000E+04	1.3942E-04	7.5000E+04	1.3273E-04	1148	3102	201
8.0000E+04	1.2673E-04	8.5000E+04	1.2132E-04	9.0000E+04	1.1640E-04	1148	3102	202
9.5000E+04	1.1191E-04	1.0000E+05	1.0780E-04	1.1000E+05	1.0019E-04	1148	3102	203
1.2000E+05	9.3717E-05	1.3000E+05	8.8131E-05	1.4000E+05	8.3256E-05	1148	3102	204
1.5000E+05	7.8960E-05	1.6000E+05	7.5142E-05	1.7000E+05	7.1725E-05	1148	3102	205
1.8000E+05	6.8645E-05	1.9000E+05	6.5853E-05	2.0000E+05	6.3310E-05	1148	3102	206
2.2000E+05	5.9051E-05	2.4000E+05	5.5591E-05	2.6000E+05	5.2731E-05	1148	3102	207
2.8000E+05	5.0330E-05	3.0000E+05	4.8290E-05	3.2000E+05	4.6538E-05	1148	3102	208
3.4000E+05	4.5018E-05	3.6000E+05	4.3691E-05	3.8000E+05	4.2523E-05	1148	3102	209
4.0000E+05	4.1490E-05	4.2000E+05	4.0607E-05	4.4000E+05	3.9828E-05	1148	3102	210
4.6000E+05	3.9138E-05	4.8000E+05	3.8525E-05	5.0000E+05	3.7980E-05	1148	3102	211
5.5000E+05	3.7396E-05	6.0000E+05	3.6870E-05	6.5000E+05	3.6163E-05	1148	3102	212
7.0000E+05	3.5520E-05	7.5000E+05	3.5167E-05	8.0000E+05	3.4840E-05	1148	3102	213
8.5000E+05	3.4742E-05	9.0000E+05	3.4650E-05	9.5000E+05	3.4552E-05	1148	3102	214
1.0000E+06	3.4460E-05	1.1000E+06	3.4440E-05	1.2000E+06	3.4410E-05	1148	3102	215
1.3000E+06	3.4490E-05	1.4000E+06	3.4360E-05	1.5000E+06	3.4340E-05	1148	3102	216
1.6000E+06	3.4310E-05	1.7000E+06	3.4290E-05	1.8000E+06	3.4270E-05	1148	3102	217
1.9000E+06	3.4250E-05	2.0000E+06	3.4230E-05	2.2000E+06	3.4520E-05	1148	3102	218
2.4000E+06	3.4810E-05	2.6000E+06	3.5100E-05	2.8000E+06	3.5390E-05	1148	3102	219
3.0000E+06	3.5680E-05	3.2000E+06	3.5800E-05	3.4000E+06	3.5910E-05	1148	3102	220
3.6000E+06	3.6030E-05	3.8000E+06	3.6140E-05	4.0000E+06	3.6260E-05	1148	3102	221
4.2000E+06	3.6290E-05	4.4000E+06	3.6320E-05	4.6000E+06	3.6360E-05	1148	3102	222
4.8000E+06	3.6390E-05	5.0000E+06	3.6420E-05	5.2000E+06	3.6290E-05	1148	3102	223
5.4000E+06	3.6160E-05	5.6000E+06	3.6040E-05	5.8000E+06	3.5910E-05	1148	3102	224
6.0000E+06	3.5780E-05	6.2000E+06	3.5670E-05	6.4000E+06	3.5560E-05	1148	3102	225
6.6000E+06	3.5450E-05	6.8000E+06	3.5340E-05	7.0000E+06	3.5230E-05	1148	3102	226
7.5000E+06	3.4590E-05	8.0000E+06	3.3940E-05	8.5000E+06	3.3640E-05	1148	3102	227
9.0000E+06	3.3330E-05	9.5000E+06	3.2960E-05	1.0000E+07	3.2590E-05	1148	3102	228
1.0500E+07	3.2210E-05	1.1000E+07	3.1820E-05	1.1500E+07	3.1450E-05	1148	3102	229
1.2000E+07	3.1080E-05	1.2500E+07	3.0630E-05	1.3000E+07	3.0180E-05	1148	3102	230
1.3500E+07	3.0010E-05	1.4000E+07	2.9830E-05	1.4500E+07	2.9400E-05	1148	3102	231
1.5000E+07	2.8960E-05	1.5500E+07	2.8630E-05	1.6000E+07	2.8300E-05	1148	3102	232
1.6500E+07	2.7880E-05	1.7000E+07	2.7450E-05	1.7500E+07	2.7360E-05	1148	3102	233
1.8000E+07	2.7260E-05	1.8500E+07	2.6730E-05	1.9000E+07	2.6200E-05	1148	3102	234
1.9500E+07	2.6120E-05	2.0000E+07	2.6040E-05	-0.	-0.	1148	3102	235
						1148	3	0
1.001 E+03	9.9917E-01	0	0	0	0	01148	3251	237
0.0	0.0	0	0	1	1	141148	3251	238
14	3					1148	3251	239
1.0	E-056.65213E-01	1.0	E+056.65213E-01	5.0	E+056.64899E-01	1148	3251	240
1.0	E+066.64620E-01	2.0	E+066.64149E-01	4.0	E+066.63355E-01	1148	3251	241
6.0	E+066.62628E-01	8.0	E+066.62018E-01	1.0	E+076.61338E-01	1148	3251	242
1.2	E+076.61045E-01	1.4	E+076.60449E-01	1.6	E+076.59929E-01	1148	3251	243
1.8	E+076.59540E-01	2.0	E+076.59196E-01			1148	3251	244
						1148	3	0
1.001 E+03	9.9917E-01	0	0	0	0	01148	3252	245
0.0	0.0	0	0	1	1	21148	3252	246
2	2					1148	3252	247
1.0	E-05	1.0	2.0	E+07	1.0		1148	3252
1.001 E+03	9.9917E-01	0	0	0	0	01148	3253	249
0.0	0.0	0	0	1	1	21148	3253	250
2	2					1148	3253	251
1.0	E-05	1.0	2.0	E+07	1.0		1148	3253
1.001 E+03	9.9917E-01	0	2	0	0	01148	4	2
0.0	0.0	0	2	0	0	01148	4	2
0.0	0.0	0	0	1	1	141148	4	2
14	2					1148	4	2

0.0	1.0-05	0	0	1	111148	4 2 261
11	2				1148	4 2 262
-1.0	.5	-.8	.5	-.6	.51148	4 2 263
-.4	.5	-.2	.5	0.	.51148	4 2 264
.2	.5	.4	.5	.6	.51148	4 2 265
.8	.5	1.0	.5		1148	4 2 266
0.0	1.0+05	0	0	1	111148	4 2 267
11	2				1148	4 2 268
-1.0	.5	-.8	.5	-.6	.51148	4 2 269
-.4	.5	-.2	.5	0.	.51148	4 2 270
.2	.5	.4	.5	.6	.51148	4 2 271
.8	.5	1.0	.5		1148	4 2 272
0.0	5.0+05	0	0	1	111148	4 2 273
11	2				1148	4 2 274
-.10000E+01	.50117E+00-.80000E+00	.50097E+00-.60000E+00	.50066E+001148	4 2 275		
-.40000E+00	.50045E+00-.20000E+00	.50025F+000.	.50005E+001148	4 2 276		
.20000E+00	.49974E+00 .40000E+00	.49953E+00 .60000F+00	.49933E+001148	4 2 277		
.80000F+00	.49902E+00 .10000E+01	.49882E+00		1148	4 2 278	
0.0	1.0+06	0	0	1	111148	4 2 279
11	2				1148	4 2 280
-.10000E+01	.50218E+00-.80000E+00	.50176E+00-.60000F+00	.50131E+001148	4 2 281		
-.40000E+00	.50089E+00-.20000E+00	.50044E+000.	.50000E+001148	4 2 282		
.20000F+00	.49957E+00 .40000E+00	.49913E+00 .60000F+00	.49869E+001148	4 2 283		
.80000E+00	.49823E+00 .10000E+01	.49779E+00		1148	4 2 284	
0.0	2.0+06	0	0	1	111148	4 2 285
11	2				1148	4 2 286
-.10000E+01	.50387E+00-.80000E+00	.50311E+00-.60000E+00	.50234F+001148	4 2 287		
-.40000E+00	.50158E+00-.20000E+00	.50081F+000.	.50003E+001148	4 2 288		
.20000E+00	.49925E+00 .40000E+00	.49846E+00 .60000E+00	.49764E+001148	4 2 289		
.80000F+00	.49682E+00 .10000F+01	.49598F+00		1148	4 2 290	
0.0	4.0+06	0	0	1	111148	4 2 291
11	2				1148	4 2 292
-.10000F+01	.50677E+00-.80000E+00	.50542E+00-.60000E+00	.50407F+001148	4 2 293		
-.40000F+00	.50275F+00-.20000F+00	.50143E+000.	.50011E+001148	4 2 294		
.20000E+00	.49873E+00 .40000E+00	.49734E+00 .60000F+00	.49589E+001148	4 2 295		
.80000E+00	.49441E+00 .10000E+01	.49283F+00		1148	4 2 296	
0.0	6.0+06	0	0	1	111148	4 2 297
11	2				1148	4 2 298
-.10000F+01	.50999E+00-.80000E+00	.50770E+00-.60000E+00	.50560E+001148	4 2 299		
-.40000E+00	.50362E+00-.20000E+00	.50177F+000.	.49993E+001148	4 2 300		
.20000F+00	.49808F+00 .40000E+00	.49628E+00 .60000E+00	.49439E+001148	4 2 301		
.80000E+00	.49246E+00 .10000E+01	.49040E+00		1148	4 2 302	
0.0	8.0+06	0	0	1	111148	4 2 303
11	2				1148	4 2 304
-.10000F+01	.51288E+00-.80000E+00	.50952E+00-.60000E+00	.50676E+001148	4 2 305		
-.40000E+00	.50434E+00-.20000E+00	.50207F+000.	.49987E+001148	4 2 306		
.20000E+00	.49766E+00 .40000E+00	.49546F+00 .60000E+00	.49314E+001148	4 2 307		
.80000F+00	.49077E+00 .10000E+01	.48818F+00		1148	4 2 308	
0.0	10.0+06	0	0	1	111148	4 2 309
11	2				1148	4 2 310
-.10000E+01	.51727E+00-.80000E+00	.51201F+00-.60000E+00	.50794F+001148	4 2 311		
-.40000E+00	.50461E+00-.20000E+00	.50168F+000.	.49908E+001148	4 2 312		
.20000F+00	.49669E+00 .40000E+00	.49435E+00 .60000E+00	.49202E+001148	4 2 313		
.80000E+00	.48969E+00 .10000E+01	.48716E+00		1148	4 2 314	
0.0	12.0+06	0	0	1	121148	4 2 315
11	2				1148	4 2 316
-.10000E+01	.52272E+00-.90000E+00	.51825E+00-.80000E+00	.51464F+001148	4 2 317		
-.60000E+00	.50898E+00-.40000E+00	.50475F+00-.20000E+00	.50129E+001148	4 2 318		
0.	.49823E+00 .20000E+00	.49556E+00 .40000E+00	.49313E+001148	4 2 319		
.60000E+00	.49085E+00 .80000E+00	.48866E+00 .10000E+01	.48654E+001148	4 2 320		
0.0	14.0+06	0	0	1	121148	4 2 321
11	2				1148	4 2 322
-.10000E+01	.52823E+00-.90000E+00	.52188E+00-.80000E+00	.51698E+001148	4 2 323		
-.60000E+00	.50982E+00-.40000E+00	.50456E+00-.20000E+00	.50048E+001148	4 2 324		
0.	.49703E+00 .20000E+00	.49431F+00 .40000E+00	.49195E+001148	4 2 325		
.60000F+00	.49005E+00 .80000E+00	.48833E+00 .10000E+01	.48688E+001148	4 2 326		
0.0	16.0+06	0	0	1	121148	4 2 327
11	2				1148	4 2 328
-.10000E+01	.53433E+00-.90000E+00	.52575E+00-.80000E+00	.51924E+001148	4 2 329		
-.60000E+00	.51024E+00-.40000E+00	.50404F+00-.20000E+00	.49939F+001148	4 2 330		

0.	.49567F+00	.20000E+00	.49288F+00	.40000E+00	.49081E+00	1148	4	2	331
.60000F+00	.48936E+00	.80000E+00	.48853F+00	.10000E+01	.48833E+00	1148	4	2	332
0.0	18.0+06	0	0	1		121148	4	2	333
12	2					1148	4	2	334
-.10000E+01	.54092E+00	-.90000E+00	.52962E+00	-.80000E+00	.52134E+00	1148	4	2	335
-.60000F+00	.51038F+00	-.40000E+00	.50327F+00	-.20000E+00	.49802E+00	1148	4	2	336
0.	.49406E+00	.20000E+00	.49126E+00	.40000F+00	.48963E+00	1148	4	2	337
.60000F+00	.48905E+00	.80000E+00	.48940E+00	.10000E+01	.49091E+00	1148	4	2	338
0.0	20.0+06	0	0	1		121148	4	2	339
12	2					1148	4	2	340
-.10000E+01	.54807E+00	-.90000E+00	.53348E+00	-.80000E+00	.52332E+00	1148	4	2	341
-.60000E+00	.51029F+00	-.40000E+00	.50221F+00	-.20000E+00	.49635E+00	1148	4	2	342
0.	.49218E+00	.20000E+00	.48958F+00	.40000E+00	.48840E+00	1148	4	2	343
.60000E+00	.48866E+00	.80000E+00	.49075E+00	.10000E+01	.49466E+00	1148	4	2	344
						1148	4	0	345
						1148	0	0	346
1.001 E+03	9.9917E-01	0	0	0		01148	7	4	347
0.0	0.0	0	0	12		11148	7	4	348
0.0	2.0 E+02	9.9917E-01	5.0	0.0	0.0	1148	7	4	349
1.0	2.0449E+01	9.9917E-01	0.0	0.0	0.0	1148	7	4	350
						1148	7	0	351
						1148	0	0	352
1.0010E+03	9.9917E-01	1	0	17		0114812102			353
0.	0.	0	0	1		2114812102			354
?	?					114812102			355
1.0000E-05	1.0000E+00	2.0000E+07	1.0000E+00			114812102			356
1.1725E+07	0.	0	2	1		3114812102			357
3	2					114812102			358
1.8000E+07	0.	1.80001E+07	1.0000E+00	2.0000E+07	1.0000E+00	00114812102			359
1.0725E+07	0.	0	2	1		4114812102			360
4	2					114812102			361
1.6000E+07	0.	1.60001E+07	1.0000E+00	1.8000E+07	1.0000E+00	00114812102			362
1.8001F+07	0.					114812102			363
9.7250F+06	0.	0	2	1		4114812102			364
4	2					114812102			365
1.4000E+07	0.	1.40001E+07	1.0000E+00	1.6000E+07	1.0000E+00	00114812102			366
1.6001F+07	0.					114812102			367
8.7250F+06	0.	0	2	1		4114812102			368
4	2					114812102			369
1.2000E+07	0.	1.20001E+07	1.0000E+00	1.4000F+07	1.0000E+00	00114812102			370
1.4001F+07	0.					114812102			371
7.7250F+06	0.	0	2	1		4114812102			372
4	2					114812102			373
1.0000E+07	0.	1.00001E+07	1.0000E+00	1.2000E+07	1.0000E+00	00114812102			374
1.2001F+07	0.					114812102			375
6.9750E+06	0.	0	2	1		4114812102			376
4	2					114812102			377
9.0000E+06	0.	9.00001E+06	1.0000E+00	1.0000E+07	1.0000E+00	00114812102			378
1.0001E+07	0.					114812102			379
6.4750E+06	0.	0	2	1		4114812102			380
4	2					114812102			381
8.0000E+06	0.	8.00001E+06	1.0000E+00	9.0000E+06	1.0000E+00	00114812102			382
9.0001E+06	0.					114812102			383
5.9750E+06	0.	0	2	1		4114812102			384
4	2					114812102			385
7.0000F+06	0.	7.00001E+06	1.0000E+00	8.0000E+06	1.0000E+00	00114812102			386
8.0001E+06	0.					114812102			387
5.4750E+06	0.	0	2	1		4114812102			388
4	2					114812102			389
6.0000E+06	0.	6.00001E+06	1.0000E+00	7.0000E+06	1.0000E+00	00114812102			390
7.0001E+06	0.					114812102			391
4.9750E+06	0.	0	2	1		4114812102			392
4	2					114812102			393
5.0000E+06	0.	5.00001E+06	1.0000E+00	6.0000E+06	1.0000E+00	00114812102			394
6.0001E+06	0.					114812102			395
4.4750E+06	0.	0	2	1		4114812102			396
4	2					114812102			397
4.0000E+06	0.	4.00001E+06	1.0000E+00	5.0000E+06	1.0000E+00	00114812102			398
5.0001E+06	0.					114812102			399
3.9750E+06	0.	0	2	1		4114812102			400

3.0000E+06	0.	2	3.0001E+06	1.0000E+00	4.0000E+06	1.0000E+00	114812102	401
4.0001E+06	0.						114812102	402
3.4750E+06	0.		0	2	1		4114812102	403
2.0000E+06	0.	2	2.0001E+06	1.0000E+00	3.0000F+06	1.0000E+00	114812102	404
3.0001E+06	0.						114812102	405
2.9750E+06	0.		0	2	1		4114812102	406
1.0000E+06	0.	2	1.0001E+06	1.0000E+00	2.0000F+06	1.0000E+00	114812102	407
2.0001E+06	0.						114812102	408
2.6250E+06	0.		0	2	1		4114812102	409
6.0000E+05	0.	2	6.0001E+05	1.0000E+00	1.0000E+06	1.0000E+00	114812102	410
1.0001E+06	0.						114812102	411
2.4250E+06	0.		0	2	1		4114812102	412
2.0000E+05	0.	2	2.0001E+05	1.0000E+00	6.0000F+05	1.0000E+00	114812102	413
6.0001E+05	0.						114812102	414
2.2250E+06	0.		0	2	1		4114812102	415
1.0000E-05	1.0000E+00	3	2.0000E+05	1.0000E+00	2.0001E+05	0.	3114812102	416
1.0010F+03	9.9917E-01						114812102	417
		1		0		0	114812102	420
							114812102	421
							114812102	422
							114812 0	423
							1148 0 0	424
							0114814102	425
							114814 0	426
							1148 0 0	427
							-1	428