

Informal Report

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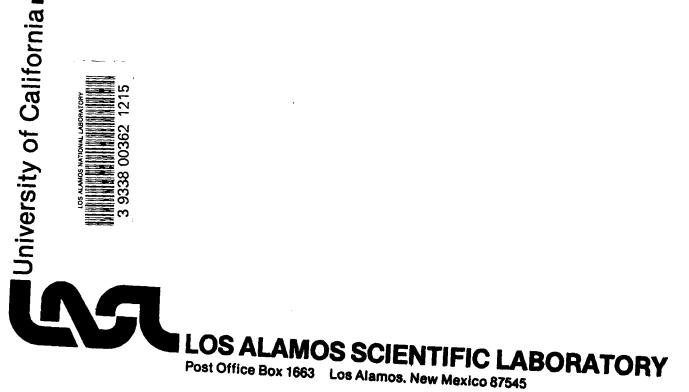
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The Costs of Electronuclear Fuel Production

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THE COSTS OF ELECTRONUCLEAR FUEL PRODUCTION

by

Theresa Flaim and Verne Loose

ABSTRACT



The Los Alamos Scientific Laboratory (LASL) proposes to study the electronuclear fuel producer (EFP) as a means of producing fissile fuel to generate electricity. The main advantage of the EFP is that it may reduce the risks of nuclear proliferation by breeding ²¹³U from thorium, thereby avoiding plutonium separation. This is a report on the costs of electronuclear fuel production based upon two designs considered by LASL. Our findings indicate that the EFP design variations considered here are not likely to result in electricity generation costs as low as the uranium fuel cycle used in the US today. At current estimates of annual fuel output (500 kg ²¹³U per EFP), the costs of electricity generation using fuel produced by the EFP are more than three times higher than generating costs using the traditional fuel cycle. Sensitivity analysis indicates that electronuclear fuel production would become cost competitive with the traditional uranium fuel cycle when U₃0₈ (yellowcake) prices approach \$1000 per pound.

INTRODUCTION

The electronuclear fuel producer (EFP) is a means of producing fissile fuel for electricitygenerating nuclear reactors. The EFP has several potential advantages over alternative methods of nuclear fuel production. First, the EFP can convert thorium to fissile material (²³³U) to fuel conventional reactors. Because thorium reserves are at least as abundant as uranium, the EFP could greatly extend the resources available for nuclear power generation.^{*} Second, by using thorium, the EFP avoids the production of plutonium^{*} and, thus, may reduce the risks of nuclear proliferation.

Electronuclear fuel production uses a linear accelerator to breed fissile material. In the design specification considered here, a high-energy proton beam (1-GeV, 300-mA) bombards a target of thorium (²³²Th). These high-energy protons interact with thorium, releasing a large number of neutrons. Some of these neutrons are absorbed by other thorium atoms, forming the uranium isotope ²³³U. After chemical processing, ²³³U can be used as fuel in conventional nuclear reactors.

This paper reports an analysis of the costs of electronuclear fuel production in two design variations: (1) as a fuel producer only, and (2) as a fuel producer, self-sustaining electricity generator. The self-sustaining EFP is considered in a fuel

Because the demand for thorium has been small in the past, thorium reserves are not well identified, However, in 1974 the Atomic Energy Commission estimated US thorium reserves to be 400 000 tons at ThO₂ prices of \$10/1b. Uranium reserves were estimated to be 340 000 tons at U₃O₈ prices of \$10/1b. Science and Public Policy Program, University of Oklahoma, "Energy Alternatives: A Comparative Analysis," prepared for the Council on Environmental Quality, et al., US Government Printing Office, Washington, DC (May 1975), pp. 6-5, 6-46.

The EFP can also convert uranium (²³⁸U) to plutonium (²³⁹Pu). However, because of the disadvantages associated with nuclear proliferation, plutonium recovery is only considered in the traditional fuel cycle to provide a basis of comparison for the EFP with fuel reprocessing.

cycle that includes fuel reprocessing. The costing of both designs presumes the sole use of thorium to produce fuel.

In order to assess the economic feasibility of the EFP, it is necessary to determine the likelihood that the EFP would result in electricity-generating costs at least equivalent to those of competing technologies. However, cost comparisons with all potential competing electricity-generating technologies are beyond the scope of this particular report. Rather, the costs of generating electricity using two types of EFP's to produce reactor fuel were compared to electricity-generating costs in the uranium fuel cycle, light-water reactor economy. both with and without fuel reprocessing. (This will be discussed in detail in the next section.) The uranium fuel cycle, light-water reactor economy without fuel reprocessing is the fuel cycle used in the US today and will also be referred to as the "traditional fuel cycle." Although these cost comparisons are based upon the use of light-water reactors, the relative costs of electronuclear fuel production and traditional uranium fuel production should remain unaffected by the reactor type so long as both processes produce equally efficient reactor fuels. Thus, relative cost comparisons based upon light-water reactors should also apply to reactors that would allow greater fuel recovery through reprocessing.

Two cost calculations were made. First, the engineering costs of electronuclear fuel production were compared with production costs in the traditional fuel cycle. Second, the present values of the costs of generating equivalent amounts of electricity were calculated for the EFP and the traditional fuel cycles. In both cost calculations, the price of U_3O_8 was varied to determine the effect of higher prices on the EFP's cost competitiveness with the traditional fuel cycle.

THE EFP AND URANIUM FUEL CYCLES

Two design variations of the electronuclear fuel producer were assessed. Case I is the fuel producer, net electricity consumer. Fuel generated in Case I is mixed with ²³⁸U and burned in a oncethrough fuel cycle. Waste heat from the EFP is dissipated by cooling towers. Case I is presented schematically in Fig. 1. Case II is the fuel producer, self-sustaining electricity generator. In Case II, 233 U is allowed to build up longer in the target/blanket to produce the 1800 MW of heat needed to generate 600 MW of electricity, assuming a 30% thermal efficiency. Fuel is mixed with thorium and reprocessed. Case II is presented schematically in Fig. 2.

The technical characteristics of the EFP (for both Cases I and II) that were assumed in calculating costs are as follows:

- The accelerator produces a 1-GeV, 300-mA proton beam.
- The accelerator's electricity to proton beam power conversion efficiency is 50%.
- The accelerator has an 80% duty factor.
- The integrated target/blanket is composed of metallic thorium spheres in a liquid sodium coolant. This "pebble-bed" design allows for the continuous loading of thorium spheres and unloading of fuel.
- Annual fuel production from the EFP is 500 kg ²³³U.

Fuel production and electricity-generating costs in Cases I and II were compared to electricitygenerating costs in the traditional uranium fuel cycle, which is presented schematically in Fig. 3. Electricity-generating costs for the Case II EFP with fuel reprocessing were compared to electricitygenerating costs in the uranium fuel cycle with fuel reprocessing (uranium and plutonium recycle). The uranium fuel cycle with fuel reprocessing is presented schematically in Fig. 4.

The costs of uranium fuel production and the costs of generating electricity in the light-water reactor economy that exists today form the basis of comparison with the EFP. Capital and operating costs associated with a 1000-MW light-water reactor scheduled for service in the late 1980's are presented in Table I. Fuel costs were estimated for the uranium fuel cycle, with and without fuel reprocessing.

Cost factors for the Case I and Case II electronuclear fuel producers are given in Tables II and III, respectively. Both cases produce 500 kg 233 U per year, which would provide more than half of the

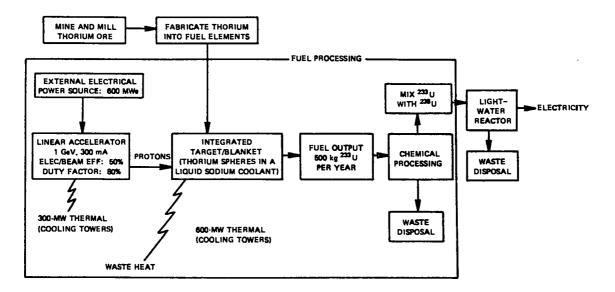


Fig. 1. Case 1 EFP: Fuel producer only, without fuel reprocessing.

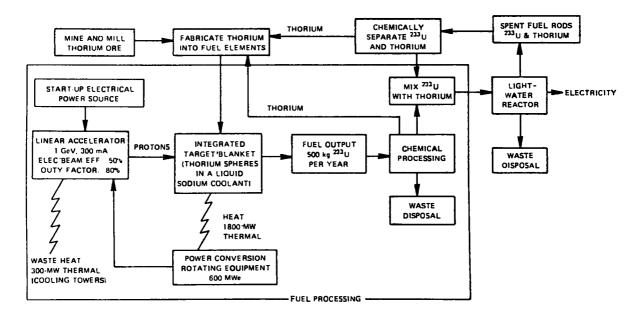


Fig. 2. Case II EFP: Fuel producer, self-sustaining electricity generator, with fuel reprocessing. In this case, ²³³U is allowed to build up in the target/blanket in order to generate the 1800 MW of heat needed to produce 600 MW of electricity (assuming a 30% thermal efficiency).

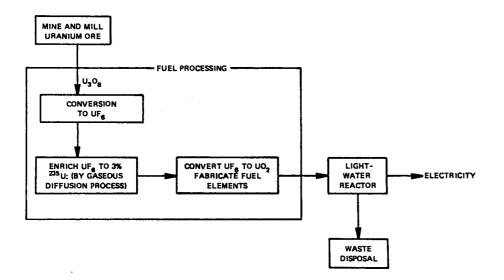


Fig. 3. Traditional uranium fuel cycle, without fuel reprocessing.

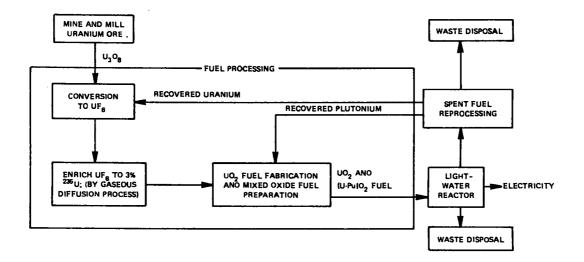


Fig. 4. Uranium fuel cycle, with fuel reprocessing (uranium and plutonium recycle).

TABLE I

CAPITAL AND OPERATING COSTS ASSOCIATED WITH A 1000 MW LIGHT-WATER REACTOR SCHEDULED FOR SERVICE IN THE LATE 1980'S (1976 DOLLARS)

Capital Cost	\$649-934	million
Annual Operating Costs		
Operation and maintenance	12	million
Fuel: Without fuel reprocessing With fuel reprocessing		million Tuillion
Total annual operating costs: Without fuel reprocessing With fuel reprocessing	. –	million million

NOTE: A range of capital cost estimates exist because of regional differences within the US. The higher capital cost estimate was used for comparison. Fuel costs per year were calculated assuming (1) a 66% plant capacity factor, (2) U₃O₈ prices of \$40 per pound and (3) enrichment costs of \$115 per separative work unit (kg SWU), also assuming that 4.2 SWU are required to enrich 1 kg of uranium to 3.2% 235 U (0.2% tails). Reprocessing costs were assumed to be \$150/kg spent fuel. Light-water reactors are assumed to have a conversion ratio (the ratio of fissile material in the spent fuel to fissile material in the original fuel) of 0.51. Thus, reprocessed uranium and plutonium will provide roughly half of the annual fuel load required for a light-water reactor (assuming 100% of the fissile material in the spent fuel is recovered).

SOURCES: Capital and operation and maintenance costs are from C. L. Rudasill, "Coal and Nuclear Generating Costs," EPRI PS-HS5-SR Special Report, Electric Power Research Institute, Palo Alto, California (April 1977), pp. 24-32.

Fuel cost estimates were obtained from a variety of sources: Saunders Miller, <u>The Economics of Nuclear and Coal Power</u>, Praeger Publishers, New York (1976), pp. 77-82; US Congress, Joint Committee on Atomic Energy, "Hearings on the Future Structure of the Uranium Enrichment Industry," Part 1, Phase 1, 93d Congress, 1st Session, US Government Printing Office, Washington, DC (July 31, 1973), pp. 39-40; Spurgeon M. Keeny, Jr., et al., "Nuclear Power Issues and Choices," Report of the Nuclear Energy Policy Study Group, Ford Foundation, Ballinger Publishing Co., Cambridge, Massachusetts (1977), pp. 109-127; Bernard L. Cohen, "The Disposal of Radioactive Wastes from Fission Reactors," <u>Scientific American</u>, (June 1977), Vol. 236, No. 6, pp. 22-23.

TABLE . II

COST FACTORS FOR THE CASE I ELECTRONUCLEAR FUEL PRODUCER (FUEL PRODUCER ONLY)

Capital cost \$800 million Annual operating costs: 60 million Electricity 60 million Other 120 million Total operating costs 120 million Construction period 10 years Useful life of plant 30 years Annual fuel production 500 kg ²³³ U Chemical Processing ^a 200 kg ²³³ U Construction period 1 million Construction period 10 years Useful life of plant 30 years	Accelerator, Target/Blanket	
Electricity 60 million Other 60 million Total operating costs 120 million Construction period 10 years Useful life of plant 30 years Annual fuel production 500 kg ²³³ U Chemical Processing ^a 1 Capital cost \$15 million Annual operating cost 1 million Construction period 10 years		\$800 million
Other 60 million Total operating costs 120 million Construction period 10 years Useful life of plant 30 years Annual fuel production 500 kg ²³³ U Chemical Processing ^a Capital cost Annual operating cost \$15 million Construction period 10 years	Annual operating costs:	
Other 60 million Total operating costs 120 million Construction period Useful life of plant 10 years 30 years Annual fuel production 500 kg ²³³ U Chemical Processing ^a 1 Capital cost Annual operating cost \$15 million 1 million Construction period 10 years	Electricity	60 million
Construction period 10 years Useful life of plant 30 years Annual fuel production 500 kg ²³³ U Chemical Processing ^a Capital cost \$15 million Annual operating cost 1 million Construction period 10 years		60 million
Useful life of plant 30 years Annual fuel production 500 kg ²³³ U <u>Chemical Processing</u> ^a <u>Capital cost</u> \$15 million Annual operating cost 1 million Construction period 10 years	Total operating costs	120 million
Useful life of plant 30 years Annual fuel production 500 kg ²³³ U <u>Chemical Processing</u> ^a <u>Capital cost</u> \$15 million Annual operating cost 1 million Construction period 10 years	Construction period	10 years
Chemical Processing [®] Capital cost \$15 million Annual operating cost 1 million Construction period 10 years		30 years
Capital cost \$15 million Annual operating cost 1 million Construction period 10 years	Annual fuel production	500 kg ²³³ U
Annual operating cost 1 million Construction period 10 years	Chemical Processing [®]	
Annual operating cost 1 million Construction period 10 years	Capital cost	\$15 million
	Construction period	10 years

The costs of chemically processing 233 U are based on the assumption that fuel can be processed by a separate processing plant. (It was assumed that processing facilities would not be constructed solely for the accelerator.) Capital and operating costs were obtained for a reprocessing plant having a capacity of 5 metric tons of spent fuel per day. The capital cost for a plant this size is \$1 billion; annual operating costs are roughly \$70 million. Proportion of annual capacity required to the EPP according to the proportion of annual capacity required to process 500 kg 233 U. (Capital and operating cost series were obtained from Gary Molen, Allied General Nuclear Services, in a personal communication on September 20, 1977.)

SOURCE: Los Alamos Scientific Laboratory, Electronuclear Fuel Production Task Force, Los Alamos, New Mexico (October 1977).

annual fuel load for a conventional light-water reactor without fuel reprocessing.

All costs are expressed in real terms using 1976 as a base year. The rationale for expressing costs in real (deflated) terms is that both general inflation and changes in relative prices should affect all cases equally because all require similar material inputs. Thus, while inflation would affect absolute costs of these cases, relative costs should remain unchanged.

DIRECT COST ESTIMATION

Engineering costs of fuel production and the present values of costs of electricity generation were estimated using the above cost data.

Engineering Cost Calculations

Engineering cost calculations allocate capital expenditures through a capital recovery factor. This calculation assumes a level, or uniform, periodic spending stream.

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TABLE III

COST FACTORS FOR THE CASE II ELECTRONUCLEAR FUEL PRODUCER (FUEL PRODUCER, SELF-SUSTAINING ELECTRICITY GENERATOR WITH FUEL REPROCESSING)

Accelerator, Target/Blanket

Capital cost Capital cost, electricity generation Annual operating cost	\$80D million 260 million 60 million
Annual operating cost, electricity generation	13 million
Construction period	10 years
Useful life of plant	30 years
Annual fuel production	500 kg ²³³ U
Chemical Processing ^a	
Capital cost	\$ 15 million
Annual operating cost	1 million
Construction period	10 years
Useful life of plant	30 years
Fuel Reprocessing	
Capital cost	\$ 30 million
Annual operating cost	2 million
Construction period	10 years
Useful life of plant	10 years 30 years
operat tite of brene	JU JEARS

^aThe costs of chemically processing ²³³U are based on the assump-tion that fuel can be processed by a separate processing plant. (It was assumed that processing facilities would not be constructed for the accelerator itself.) Capital and operating costs were obtained for a reprocessing plant having a capacity of 5 metric tons of spent fuel per day. The capital cost for a plant this size is \$1 billion; annual operating costs are roughly \$70 million. Pro-portions of these costs were assigned to chemical processing and fuel reprocessing according to the proportion of annual capacity required for each activity. (Capital and operating cost estimates were obtained from Gary Molen, Allied General Nuclear Services, in a personal communication on September 20, 1977.)

SOURCE: Los Alamos Scientific Laboratory, Electronuclear Fuel Production Task Force, Los Alamos, New Mexico (October 1977).

Engineering costs of electronuclear fuel production and uranium fuel production are presented in Table IV. These calculations indicate that electronuclear fuel production costs would be approximately \$400/g ²³³U. This is more than 10 times higher than current costs of ²³⁵U production. Sensitivity analysis indicates that electronuclear fuel production would become cost competitive with traditional uranium fuel production if U308 prices approach \$1000 per pound.

Present Value of Cost Calculations

In addition to calculating the engineering costs of fuel production, the present values of costs of generating equivalent amounts of electricity were calculated for the Case I and Case II EFP's and for the uranium fuel cycle, with and without

TABLE IV

ENGINEERING COSTS OF ELECTRONUCLEAR FUEL PRODUCTION AND THE TRADITIONAL URANIUM FUEL CYCLE (1976 DOLLARS)

	Dollars/g 233U or 235U
Electronuclear Fuel Production [®]	
Case I (fuel producer only)	\$415
Case II (self-sustaining electricity generator)	375
Traditional Uranium Fuel Cycle ^b	
U ₃ 0 ₈ € \$40/1Ъ.	40
U ₃ 0 ₈ • \$100/1Ъ.	65
U ₃ 0 ₈ @ \$200/1b.	105
U ₃ 0 ₈ e \$500/1b.	225
U ₃ 0 ₈ e \$1000/1b.	425

^aProduction costs for electronuclear fuel production are based upon the following information:

the following information: • <u>Case I EFP</u>: 1) annual capital costs, \$86 million based upon a 10% compound rate of interest, a 30-year useful life, and a capital recovery factor of 10.6% per year, 2) annual operating costs, \$120 million, and 3) annual fuel production, 500 kg ²³³U. • <u>Case II EFP</u>: 1) annual capital costs, \$114 million, based upon a 10% compound rate of interest, a 30-year useful life and a capital recovery factor of 10.6% per year, 2) annual operating costs, \$73 million and 3) annual fuel production, 500 kg ²³³U.

^bTraditional fuel cycle costs are based upon the following information: 4.3 kg SMU are required to enrich one kg U to 3^{k} ²³⁵U at a 0.2^k tails assay; 12 lb of U₃Og are required for 1 kg enriched U at the second se 0.2% tails assay; enrichment costs are \$115 kg SMU; conversion to UFG is \$3.33/kg; fabrication is \$90/kg; spent fuel storage and di posal costs are \$100/kg. U30g prices were varied as shown in the table. All other fuel costs remain constant. dis-

NOTE: A 1000 MN light-water reactor operating at a 66% plant capacity factor requires an annual fuel load of 700 kg 233 U or 235 U.

SOURCES: Los Alamos Scientific Laboratory, Electronuclear Fuel Production Task Force, Los Alamos, New Mexico (October 1977); US Congress, Joint Committee on Atomic Energy, "Hearings on the Puture Structure of the Uranium Enrichment Industry," Part 1, Phase 1, 93d Congress, 1st Session, US Government Printing Office, Washington, DC (July 31, 1973), pp. 39-40; Spurgeon M. Keeny, Jr., et al., "Nuclear Power Issues and Choices," Report of the Nuclear Energy Policy Study Group, Ford Foundation, Ballinger Publishing Co., Cambridge, Massachusetts (1977), pp. 109-127; Saunders Miller, The Economics of Nuclear and Coal Power, Praeger Publishers, New York (1976), pp. 77-82.

fuel reprocessing.* Included among the costs for the present value calculations are the costs of mining and milling ore, fuel processing, electricity generation, waste handling, and (where included) fuel reprocessing. Electricity transmission costs

$$PV = \sum_{t=1}^{n} C_{t} \left[\frac{1}{(1+r)^{t}} \right],$$

where t = year, including both the construction phase and the plant's useful life,

 $C_t = cost incurred in year t, and$

r = annual discount rate.

^{*}The formula for calculating present value for discrete-time discounting is:

are excluded; it was assumed that these costs are equal for all cases. Also excluded are research and development costs associated with the EFP.*

Two factors are important in calculating the present value of costs: (1) the total costs of each technology, and (2) the time profile of expenditures.^{**} A complete estimate of total costs is obtained by insuring that all costs associated with each technology are included. The timing of expenditures is represented by discounting. Because the technologies may involve different spending streams, it is necessary to standardize them. Discounting achieves this by applying decreasing weights to expenditures of greater futurity. Consequently, calculating the present value of costs yields a single number for each technology, which is then used for comparison.

A discount rate of 10% was used to calculate the present value of costs. This discount rate reflects an estimated average real rate of return on capital (before taxes) in the private sector.

Life cycle costing of electricity generation using the EFP or the uranium fuel cycle is necessary for three reasons. First, the cases evaluated will require spending over a 30- to 40-year time horizon. Consequently both total costs and the timing of expenditures are important. Second, fuel cycle and electricity generation costs must be considered because the fuel cycle will determine the number of light-water reactors the EFP can supply, a crucial factor affecting its economic feasibility. Third, the Case II EFP includes fuel reprocessing; it is necessary to calculate electricity-generating costs in order to determine how fuel reprocessing in the EFP and uranium fuel cycles compares to the traditional uranium fuel cycle without fuel reprocessing.

The present values of the costs of generating electricity in the uranium and EFP fuel cycles (with and without fuel reprocessing) are presented in Table V. Without fuel reprocessing, the present values of the costs of generating electricity using either the Case I or Case II EFP to produce reactor fuel are nearly three times greater than the present value of costs of the traditional fuel cycle. The price of U_3O_8 was assumed to be \$40 per pound in the base case. Sensitivity analysis indicates that electronuclear fuel production would result in electricity-generating costs roughly equal to the uranium fuel cycle if U_3O_8 prices reach \$1000 per pound.

With fuel reprocessing, the present value of the costs of generating electricity with the EFP is twice as high as the uranium fuel cycle. At U_3O_8 prices of \$1000 per pound, the uranium fuel cycle with reprocessing is still slightly cheaper than electronuclear fuel production.

Light-water reactors have a conversion ratio of 0.51 (the ratio of fissile material in the spent fuel to the fissile material in the original fuel). Thus, roughly half a light-water reactor's annual fuel load can be provided by fuel reprocessing.^{*} Reactors with higher conversion ratios allow greater fuel recovery. Evaluating the effect of introducing higher converter reactors on the economic feasibility of electronuclear fuel production requires comparing electricity-generating costs of both the EFP and uranium fuel cycles when both produce fuel that is burned in higher converter reactors and fuel is reprocessed. However, so long as ²³⁵U and ²³³U are equally efficient fuels for reactors with higher

A complete assessment of the EFP should include research and development costs. However, the difficulties of predicting these costs necessitated their exclusion.

^{**} The importance of both factors can be illustrated by comparing the present value of supplying electricity to the accelerator by (1) purchasing electricity, and (2) using waste heat from the target/ blanket to generate electricity directly. Purchased electricity is assumed to cost \$60 million per year, or \$1.8 billion over the accelerator's 30-year operating life. The costs of generating electricity, as shown in Fig. 2, include capital costs of \$260 million and \$13 million per year for operating costs. Thus, total capital and operating costs for generating electricity directly are \$650 million.

The present value (at a 10% discount rate) of the costs of purchased electricity is \$218 million compared with \$209 million for self-generation. Thus, while the total costs of purchased and selfgenerated electricity differ by \$1 billion, the present values of costs differ by only \$9 million.

This is true, assuming that 100% of the fissile material in the spent fuel is recovered. Bernard L, Cohen, "The Disposal of Radioactive Wastes from Fission Reactors," <u>Scientific American</u> (June 1977), Vol. 236, No. 6, pp. 22-23.

TABLE V

PRESENT VALUES OF ELECTRICITY GENERATING COSTS FOR THE URANIUM FUEL CYCLE AND THE CASE I AND CASE II ELECTRONUCLEAR FUEL PRODUCERS (STANDARDIZED TO A 1000-MW LIGHT-WATER REACTOR)

		Sensitivity Analysis ^b				
	Base Case ^a	\$100/15 U308	\$2D0/15 U_308	\$500/15 U ₃ 0 ₈	\$1000/15 U308	
		(in millions of 1976 dollars)				
Without Fuel Reprocessing						
Traditional uranium fuel cycle	\$ 740	\$ 820	\$ 950	\$1300	\$2000	
Case I EFP (fuel producer only)	1980	1980	1980	1980	1980	
Case II EFP (self-sustaining electricity generator)	1970	1970	1970	1970	1970	
With Fuel Reprocessing ^C						
Uranium fuel cycle with uranium and plutonium recycle	720	760	820	1000	1300	
Case II EFP (self-sustaining electricity generator)	1480	1480	1480	1480	1480	

^aTables I, II, and III list the base cost factors for the light-water reactor economy that exists today, the Case I EFP, and the Case II EFP, respectively. The present values of costs of the Case I and Case II EFP's include the capital costs of the light-water reactors fueled by the EFP's.

^bAll other costs are identical to the base case. Because ²³⁸U is mixed with ²³³U in the Case I EFP, the present values of generating costs for the Case I EFP should also rise as U_3O_8 prices increase. However, the purpose of the sensitivity analysis was to show how the costs of electricity generation with the EFP compare to generation costs in the uranium fuel cycle at varying U_3O_8 prices. The reader should be aware that the present values of costs for the Case I EFP exclude higher fuel costs that would result from higher U_3O_8 prices.

^CThe Nuclear Regulatory Commission estimated fuel reprocessing costs for the uranium-plutonium cycle to be \$150/kg spent fuel; this estimate was used in the above calculations. Reprocessing costs are difficult to predict at the present time and some authors feel that \$200-\$400/kg spent fuel is more realistic. Spurgeon M. Keeny, Jr., et al., "Nuclear Power Issues and Choices," Report of the Nuclear Energy Policy Study Group, Ford Foundation, Ballinger Publishing Company, Cambridge, Massachusetts (1977), p. 326. Reprocessing costs for ²³³U were estimated to be roughly \$110/kg spent fuel. Fuel is not reprocessed in the Case I EFP.

NOTE: All calculations assume a 10-year construction period and that facilities (LMR's and EFP) have a useful life of 30 years. Present values of cost calculations assume the following percentages of total capital costs are spent in years 1 through 10 (construction phase), respectively: 5, 10, 10, 15, 15, 10, 10, 5, and 5%. The costs of the initial light-water reactor core loading (3000 kg ²³⁵U or ²³³U) are excluded in the above calculations. A 1000-MW light-water reactor operating at a 66% capacity factor consumes 22 500 kg of fuel per year, or roughly 700 kg ²³⁵U or ²³³U (assuming 3.2% enrichment). Saunders Miller, <u>The Economics of Nuclear and Coal Power</u>, Praeger Publishers, New York (1976), p. 78.

conversion ratios, the results of these cost calculations based upon light-water reactors should also apply to reactors that would allow greater fuel recovery through reprocessing. The above analysis indicates that the electronuclear fuel producer in the design variations considered here is unlikely to result in electricity generation costs that are lower than the uranium fuel cycle, even if $U_{3}O_{8}$ prices increase dramatically.

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