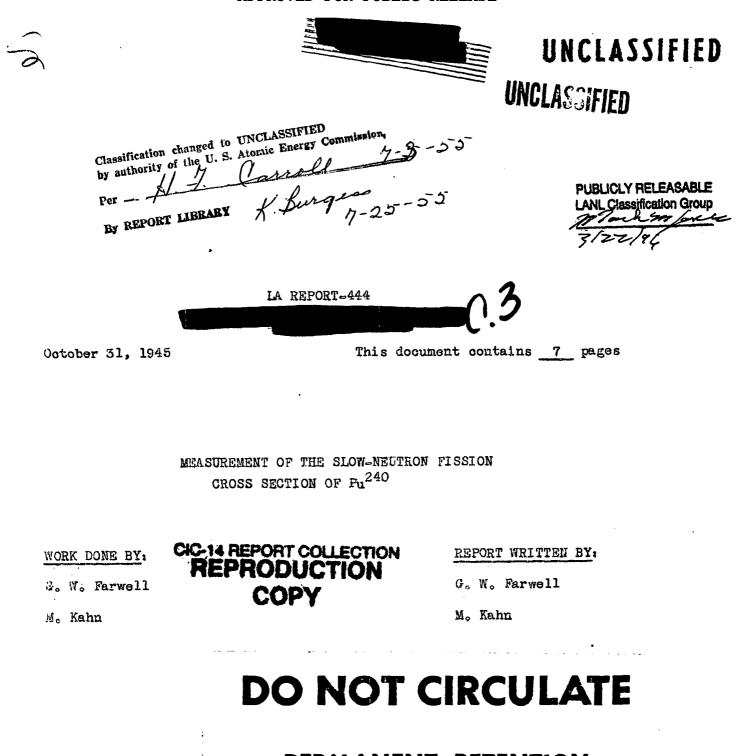
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ABSTRACT

The ratio of the slow-neutron fission cross sections of Pu^{240} and

Pu²³⁹ is

 $\frac{\sigma_{\rm f}(40)}{\sigma_{\rm f}(49)} = 0.09 \pm 0.33$







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MEASUREMENT	OF	THE	SLOW-NEUTRON	FISSION	CROSS	SECTION	OF	Pu ²⁴⁰	

In March, 1945, two samples of Pu containing different amounts (0.0 percent and 0.65 percent) of Pu^{240} were compared by alpha count, and again by fission count when placed in the same slow-neutron flux. Results were not conclusive, but the indioation was that Pu^{240} was an alpha emitter of shorter half-life than Pu^{239} , or that Pu^{240} had a smaller slow-neutron cross section, or both.

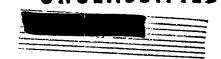
Pu of higher Pu^{240} content has become available, and several determinations of the half-life of Pu^{240} have been made. The experiment has been repeated, using more highly enriched Fu, in order to make an approximate determination of the fission cross section of Pu^{240} .

Two pairs of samples were prepared, samples "C-24," containing about 0.05 percent Pu²⁴⁰, and samples "CW-2," (of Hanford reirradiated material) containing about 4 percent Pu²⁴⁰. The samples were prepared as follows:

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Solutions of plutonium 1N in HCl, containing approximately one microgram of glutonium per ml, were prepared and stored in quartz vessels. Quartz-distilled water and quartz-distilled HCl used were furnished by Wickers. About twenty microliters of the plutonium solution were spread in the form of fine droplets over a 2-cm-diameter circle on a platinum disc. These droplets were then evaporated under a heat lamp. This process was repeated until the desired amount of plutonium was deposited on the platinum disc. At frequent intervals during the process the platinum disc was ignited in a flame. The total volume evaporated per plate was of the order of one ml, and the Fu deposited was of the order of one-half µg. The resulting plates were practically invisible, and completely transparent to fission fragments.

 $\sigma_{f}(40)/\sigma_{f}(49)$ can be computed as follows from a comparison of two samples 1 and 2 of different Pu²⁴⁰ content: UNCLASSIFIED



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$$\sigma_{f}(40)/\sigma_{f}(49) = (1/F_{2}) (N_{2}/N_{1}) (S_{2}/S_{1}) (a_{1}/a_{2}) - (1-F_{2})/\overline{F_{2}}$$

where

 F_2 is the ratio of Pu²⁴⁰/Pu in sample 1; N₁ and N₂ are the fission counting rates of samples 1 and 2, respectively, when placed in the same slow-neutron flux;

 S_1 and S_2 are the specific alpha activities of materials 1 and 2 from which the samples are made;

 a_1 and a_2 are the alpha counting rates of samples 1 and 2; and where material 1 has a Pu^{240}/Pu ratio F_1 which is essentially zero, as is the case for the "C-24" samples used in the experiment.

 F_1 and F_2 were determined by spontaneous fission measurements and were found to be $F_1 = 0.0005 \pm 0.0001$ (C-24) and $F_2 = 0.0404 \pm 0.002$ (CW-2).

The ratio N_2/N_1 was determined by placing two samples of approximately equal weight (~ 0.7 µg in one determination and ~ 0.5 µg in a second determination) in a comparison chamber, irradiating the samples with slow neutrons, and counting the fissions occurring in each sample. The comparison chamber was an argon-filled flat cylindrical steel chamber in which the samples were mounted back to back on a central electrode at a potential of about -500 volts. Collecting electrodes opposite each sample and about 1 cm distant from the central electrode were connected to twin linear amplifiers and counting circuits. The chamber was placed in the graphite column of the water boiler at Omega, at a point at which the cadmium ratio was about 2000. With the water boiler operating at 1 to 2 KW, counting rates of about 20,000 fissions per minute were obtained. Fission counts were taken at three different biases on the fission plateau and the extrapolated counting rates were used in the calculations. In order to avoid any errors due to differences in chamber geometry or amplifier characteristics two runs were made with each pair of samples, the second run being made with sample positions interchanged.

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Alpha counts were made directly in an argon chamber connected to a fast amplifier and scale of 256.

The specific alpha activities S_1 and S_2 were calculated on the basis of half-lives of 24,400 and 6,300 years for Pu^{239} and Pu^{240} , respectively, corresponding to specific activities of 136,000 and 529,000 alpha disintegrations per minute per microgram¹).

	First Determination	Second Determination
Sample 1	C-24-1	C-24-3
Sample 2	CW2-1	CW2=3
F1	0.0005	0.0005
F2	0.0404 + 0.002	0.0404 + 0.002
sl	136,200 disintegrations/min x μ g	136,200
S2	151,900 " " "	151,900
Nl	21,420 + 70 fission counts/min	24,700 + 70
N ₂	19,410 - 70 "	20,050 + 70
al	$51,780 \pm 100$ alpha counts/min	34,730 + 100
^a 2	54,130 <u>+</u> 100 " "	32 ,780 <u>+</u> 100

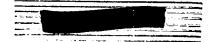
Experimental data and calculated results are as follows:

$$\sigma_{\rm f}(40)/\sigma_{\rm f}(49) = 0.18$$

 $\sigma_{f}(40)/\sigma_{f}(49) = -0.01$

Average value for $\sigma_f(40)/\sigma_f(49) = 0.09 \pm 0.33$

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Scaborg, in a letter to Allison, has reported that similar experiments performed at Chicago gave values for $\sigma_{f}(40)$ of zero and $(1/6)\sigma_{f}(49)$; these results are in reasonable agreement with the results obtained here.

The results do not take into account the possible contribution of Pu^{241} to the slow neutron fission counts. This contribution is probably small, as may be seen from the following reasoning:

Suppose

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 $\sigma_{f}(40) \simeq (1/10)\sigma_{f}(49)$ $\sigma_{f}(41) = \sigma_{f}(49)$ $\sigma_{c}(40) = \sigma_{c}(49)$

The emount of 40 in the reirradiated 49 is about 4 percent. Then the amount of 41 in 40 would be about 2 percent, assuming no decay of 41. The increment in the experimental value of $\sigma_f(40)/\sigma_f(49)$ caused by the presence of 41 would then be of the order of 2 percent, making no observable difference in the experimental results. However, assignment of values to $\sigma_0(40)$ and $\sigma_f(41)$ which are several times the corresponding cross sections of 49 would indicate that $\sigma_f(40)$ is zero and the apparent fission of 40 is due entirely to 41.

The uncertainty in the half-life of Fu^{240} (about 10 percent) introduces the largest single error in the experiment. A direct determination of the specific activity S_2 of the enriched material, accurate to $\pm 1/2$ percent, would bring the error due to the uncertainty in S_2 down to the level of the other experimental errors involved, and bring the overall probable error in $\sigma_f(40)/\sigma_f(49)$ down to \pm 0.2. Such an analysis is difficult with the very small amount of reirradiated material available for the experiment. A more precise determination of the half-life of Fu^{240} , if it should be made, will improve the accuracy of the experiment. A change of ± 5 percent in the

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half life of Pu^{240} would result in a change of -0.15 in $\sigma_f(40)/\sigma_f(49)$. Availability of Pu considerably higher in Pu^{240} content would of course make a more precise determination of $\sigma_f(40)/\sigma_f(49)$ by this method possible.

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