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**Neutron Production from (α ,n) Reactions and
Spontaneous Fission in ThO_2 , UO_2 , and
 $(\text{U},\text{Pu})\text{O}_2$ Fuels**

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Neutron Production from (α, n) Reactions and Spontaneous Fission in ThO_2 , UO_2 , and $(\text{U}, \text{Pu})\text{O}_2$ Fuels



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NEUTRON PRODUCTION FROM (α ,n) REACTIONS AND SPONTANEOUS FISSION
IN ThO_2 , UO_2 , AND $(\text{U},\text{Pu})\text{O}_2$ FUELS

by

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ABSTRACT

Available alpha-particle stopping cross-section and $^{17,18}\text{O}(\alpha,\text{n})$ cross-section data were adjusted, fitted, and used in calculating the thick-target neutron production function for alpha particles below 10 MeV in oxide fuels. The spent UO_2 function produced was folded with actinide decay spectra to determine (α,n) neutron production by each of 89 actinides. Spontaneous-fission (SF) neutron production for 40 actinides was calculated as the product of $\bar{v}(\text{SF})$ and SF branching-fraction values accumulated or estimated from available data. These contributions and total neutron production in spent UO_2 fuel are tabulated and, when combined with any calculated inventory, describe the spent UO_2 neutron source. All data are tabulated and methodology is described to permit easy extension to specialized problems.

I. INTRODUCTION

Neutron sources are present in reactor fuel from the spontaneous-fission (SF) decay of actinide nuclides and from the interaction of their decay alpha particles with low- and medium-Z nuclides in (α,n) reactions. The (α,n) source in oxide fuels is dominated by reactions with ^{17}O and ^{18}O , which are present in NATO in 0.038 and 0.204 atom percent abundancies, respectively.

The probability of neutron production by an alpha particle emitted at energy E_α in the fuel is given by the thick-target neutron production function $P(E_\alpha)$, which we have evaluated for four fuel compositions--clean ThO_2 thermal

reactor fuel, clean and spent UO_2 thermal reactor fuel, and clean $(\text{U}, \text{Pu})\text{O}_2$ fast reactor fuel. The (α, n) neutron production function has been evaluated at the Hanford Engineering Development Laboratory (HEDL) by Ombrellaro and Johnson for alpha particles in FFTF fuel;¹ however, $P(E_\alpha)$ has not been calculated for the fuels of interest here, and the change in $P(E_\alpha)$ with exposure has not been evaluated. We have employed the methodology and data used in the HEDL work¹ with minor exceptions in data and energy range of calculation.

The equations describing (α, n) and SF neutron production and the data quantities used in the calculations are given in Sec. II. The available data sources and adjustments made to the data are described in Sec. III. Details of the (α, n) calculations are briefly discussed in Sec. IV. Resulting (α, n) , SF, and total neutron production values are given in Sec. V for each of a variety of actinide nuclides produced in reactor fuels.

Selected results of these calculations have been reported previously.²⁻⁵

II. THEORY

The slowing and stopping of alpha particles in a material are described by the material's alpha-particle stopping power,

$$SP(E) = - \frac{dE}{dx} , \quad (1)$$

which gives the energy-dependent energy loss of alpha particles of energy E per unit path length x .⁶ The energy loss of an alpha particle of initial energy E_α in traveling a distance X can be determined from the stopping power as

$$\Delta E = E_\alpha - E'_\alpha = \int_0^X \left(- \frac{dE}{dx} \right) dx . \quad (2)$$

Similarly, the distance traveled in slowing from E_α to E'_α is

$$X = \int_{E'_\alpha}^{E_\alpha} \alpha \frac{1}{\left(\frac{dE}{dx} \right)} dE = \int_{E'_\alpha}^{E_\alpha} \frac{1}{\left(- \frac{dE}{dx} \right)} dE . \quad (3)$$

Neutrons may be produced within the material by (α, n) reactions with nuclide i , which has atom density N_i and microscopic (α, n) cross section $\sigma_i(E)$. The probability of (α, n) interaction with nuclide i by an alpha particle of energy E traveling from x to $x + dx$ is

$$N_i \sigma_i(E) dx = \frac{N_i \sigma_i(E) dE}{\left(\frac{dE}{dx}\right)} . \quad (4)$$

The probability of (α, n) interaction with nuclide i by an alpha particle in lieu of slowing from E_α to E'_α is then

$$p_i(E_\alpha, E'_\alpha) = \int_{E_\alpha}^{E'_\alpha} \frac{N_i \sigma_i(E) dE}{\left(\frac{dE}{dx}\right)} = \int_{E'_\alpha}^{E_\alpha} \frac{N_i \sigma_i(E) dE}{\left(-\frac{dE}{dx}\right)} . \quad (5)$$

The probability of (α, n) interaction with nuclide i by an alpha particle prior to stopping in the material is given by the thick-target neutron production function

$$P_i(E_\alpha) = \int_0^{E_\alpha} \frac{N_i \sigma_i(E) dE}{\left(-\frac{dE}{dx}\right)} . \quad (6)$$

In addition to that of the above definition of Eq. (1), a variety of quantities are referred to as "stopping powers" or often alternately "stopping cross sections." These include (typically without explicit regard to sign) the quantities $\frac{dE}{dx} = \frac{dE}{d(\rho x)} = \frac{dE}{\rho dx}$ ⁷, $\frac{dE}{Z^2 dx}$ ⁸, $\rho \frac{dE}{dx}$, and $\frac{dE}{N dx}$ ⁹. Here x is material thickness (mg/cm^2), Z is atomic number, ρ is material density (g/cm^3), and N is the total atom density of the material ($atoms/cm^3$). The last quantity is also called the stopping cross section,

$$\epsilon(E) = -\frac{1}{N} \frac{dE}{dx} , \quad (7)$$

a notation adopted here. Equations above defining p_i and P_i may now be written in terms of ϵ as

$$p_i(E_\alpha, E'_{\alpha}) = \frac{N_i}{N} \int_{E'_{\alpha}}^{E_\alpha} \frac{\sigma_i(E)}{\epsilon(E)} dE \quad . \quad (8)$$

and

$$p_i(E_\alpha) = \frac{N_i}{N} \int_0^{E_\alpha} \frac{\sigma_i(E)}{\epsilon(E)} dE \quad . \quad (9)$$

Note that p_i and P_i are related by

$$p_i(E_\alpha, E'_{\alpha}) = p_i(E_\alpha) - p_i(E'_{\alpha}) \quad . \quad (10)$$

The stopping cross section $\epsilon(E)$ of a material composed of J elemental constituents may be calculated using the Bragg-Kleeman¹⁰ relationship, which may be written as

$$\epsilon(E) \approx \frac{1}{N} \sum_{j=1}^J N_j \epsilon_j(E) \quad , \quad (11)$$

where

$$N = \sum_{j=1}^J N_j \quad . \quad (12)$$

The accuracy of the approximation of Eq. (11) will be discussed in Sec. III.

A fraction of the decays of nuclide k within the material may be by alpha-particle emission. This fraction F_k^α of alpha decays may occur with the emission of one of L possible alpha-particle energies. The intensity $f_{k\ell}^\alpha$ is the fraction of all decays of nuclide k resulting in an alpha particle of energy $E_{k\ell}$, and

$$F_k^\alpha = \sum_{\ell=1}^L f_{k\ell}^\alpha \quad . \quad (13)$$

The fraction of nuclide k decays resulting in (α, n) neutron production in a thick-target material containing I nuclides with (α, n) cross sections is thus

$$R_k(\alpha, n) = \sum_{\ell=1}^L f_{k\ell}^\alpha \sum_{i=1}^I p_i(E_{k\ell}) . \quad (14)$$

The SF of an actinide nuclide k is accompanied by the emission of an average \bar{v}_k^{SF} neutrons. The SF activity A_k^{SF} of nuclide k , having atom density N_k , is

$$A_k^{SF} = \lambda_k^{SF} N_k . \quad (15)$$

Here, λ_k^{SF} is the SF decay constant defined by

$$\lambda_k^{SF} = \ln 2 / T_{1/2}^k(SF) , \quad (16)$$

where $T_{1/2}^k(SF)$ is the SF half-life of nuclide k . SF is typically only one of M modes of decay; the total activity due to nuclide k is

$$A_k = \lambda_k N_k = \sum_{m=1}^M A_k^m , \quad (17)$$

where λ_k is the total decay constant of nuclide k ,

$$\lambda_k = \sum_{m=1}^M \lambda_k^m = \ln 2 / T_{1/2}^k , \quad (18)$$

and $T_{1/2}^k$ is the total half-life of nuclide k . The fraction of nuclide k decays by SF is given by the SF branching fraction

$$F_k^{SF} = A_k^{SF} / A_k = \lambda_k^{SF} / \lambda_k = T_{1/2}^k / T_{1/2}^k(SF) . \quad (19)$$

The average number of SF neutrons emitted per decay (by any mode) of nuclide k is then

$$R_k(SF) = F_k^{SF} \bar{v}_k(SF) . \quad (20)$$

The total number of neutrons, on the average, emitted due to (α, n) reactions and SF is

$$R_k = R_k(\alpha, n) + R_k(SF) . \quad (21)$$

The total neutron source S from (α, n) reactions and SF within a material containing K pertinent radionuclides is then

$$S = \sum_{k=1}^K \lambda_k N_k R_k . \quad (22)$$

The evaluation of the quantities $R_k(\alpha, n)$, $R_k(SF)$, and R_k for a number of actinide nuclides is described in the following sections.

III. DATA

The data quantities required to compute the neutron production fractions $R_k(\alpha, n)$ and $R_k(SF)$ for each of the four fuels of interest include the following.

- For each major elemental constituent j of the material: N_j , the atom density; and $\epsilon_j(E)$, the alpha-particle stopping cross section.
- For each nuclide i within the material having an (α, n) cross section: N_i , the atom density; and $\sigma_i(E)$, the microscopic (α, n) cross section.
- For each nuclide k decaying by alpha decay: $f_{k\ell}^\alpha$, the intensity for emission of each L alpha particles; and $E_{k\ell}$, the energy of each of L alpha particles.
- For each nuclide k decaying by SF: F_k^{SF} , the SF branching fraction; and $\bar{v}_k(SF)$, the average number of neutrons emitted per SF.

A. Stopping Cross Section $\epsilon(E)$

Densities of each constituent of each fuel type are given in Table I. The fuel composition of UO_2 LWR fuel is given for clean and spent conditions for the evaluation of the effect of exposure-dependent fuel composition on stopping cross section ϵ ; here, $_{41}Nb$ and $_{59}Pr$ represent the low- and high-mass fission products, respectively. Concentrations of $_{93}Np$, $_{95}Am$, and $_{96}Cm$ are given for the spent UO_2 fuel, although the minor contributions to ϵ from these nuclides are included as plutonium. Elements contributing to the material stopping cross sections are thus O, Nb, Pr, Th, U, and Pu.

A bibliography of experimental and theoretical stopping-power references by Anderson¹¹ notes that some 900 papers have been published on the subject of ion energy loss in matter. Anderson, noting the observation by Bichsel¹² that stopping powers measured by different groups often did not agree within stated uncertainties, was unable to resolve discrepancies after careful analysis and cautioned that stopping-power data sources should be selected carefully. We have chosen as the major stopping cross-section data source the comprehensive volume edited by Ziegler,¹³ which gives tabulated alpha stopping cross-section values and functional fits for elements in the range $1 \leq Z \leq 92$.

No values of the alpha-stopping cross section for plutonium were identified, although values for plutonium compounds were found.⁷ Northcliffe and Schilling⁸ have tabulated values of the stopping power dE/dx for $Z \leq 92$. They have shown graphically, for each Z including $Z = 94$, the energy-dependent ratio $(dE/dx)_Z : (dE/dx)_{A\alpha}$. In order to form a stopping cross section for plutonium consistent with the data of Ziegler,¹³ we have used the stopping power ratio of Ref. 8 in the expression

$$\epsilon_{Pu} = \epsilon_u \frac{A_{Pu}}{A_U} \left[(dE/dx)_{Pu} : (dE/dx)_{A\alpha} \frac{(dE/dx)_{A\alpha}}{(dE/dx)_U} \right] , \quad (23)$$

where all quantities enclosed in brackets [] were taken from Ref. 8. Values used and produced in this calculation are given in Table II.

Fourth-degree polynomial functions of the form

$$\ln \epsilon = C_0 + C_1 \ln E + C_2 \ln^2 E + C_3 \ln^3 E + C_4 \ln^4 E \quad (24)$$

TABLE I
PROPERTIES OF OXIDE FUELS

	Thermal Reactor Fuels			Fast Reactor Fuel (U,Pu)O ₂ Clean
	UO ₂ Clean	UO ₂ Spent	ThO ₂ Clean	
Fuel Density (g/cm ³)	9.95	9.95	9.17	9.62
Exposure GWd/t	0	34	0	0
Atom Densities (atoms/b-cm)				
⁸⁰ N _A T	0.04372	0.04372	0.04184	0.04215
¹⁶ O	0.04361	0.04361	0.04174	0.04205
¹⁷ O	1.6614-5	1.6614-5	1.5899-5	1.6017-5
¹⁸ O	8.9189-5	8.9189-5	8.5354-5	8.5986-5
⁴¹ Nb	0	7.893-4	0	0
⁵⁹ Pr	0	7.893-4	0	0
⁹⁰ Th	0	0	0.02025	0
⁹² U	0.02186	0.02085	6.724-4	0.01887
⁹³ Np	0	1.043-5	0	0
⁹⁴ Pu	0	2.037-4	0	0.002634
⁹⁵ Am	0	5.692-6	0	0
⁹⁶ Cm	0	1.131-6	0	0

TABLE II

DATA OF NORTHCLIFFE AND SCHILLING^a AND ZIEGLER^b USED IN
CALCULATING THE ALPHA PARTICLE STOPPING CROSS SECTION OF PLUTONIUM

E_{α} MeV	Stopping Power Ratios and Values from Northcliffe and Schilling				ε(E) Stopping Cross Section	
	(dE/dx) _{Pu}		(MeV/mg/cm ²)	(dE/dx) _{Pu}	eV/(10 ¹⁵ atoms/cm ²)	
	(dE/dx) _{Aα}	(dE/dx) _{Aα}		(dE/dx) _U		
0.100	0.150	0.752	0.135	0.837	75.80	63.74
0.320	0.188	1.219	0.243	0.942	139.93	132.48
0.500	0.214	1.317	0.286	0.986	165.64	164.08
0.805	0.235	1.299	0.312	0.978	178.59	175.40
1.281	0.256	1.170	0.307	0.977	166.77	163.72
2.402	0.291	0.904	0.269	0.978	129.15	126.86
4.003	0.322	0.682	0.223	0.982	100.57	99.23
6.404	0.350	0.512	0.183	0.978	78.65	77.29
10.007	0.382	0.379	0.148	0.980	60.67	59.71
16.010	0.418	0.270	0.114	0.991	47.09	46.90
24.016	0.448	0.200	0.090	1.000	37.01	37.18
48.031	0.490	0.118	0.059	0.983	23.64	23.35

^aNorthcliffe and Schilling, Nucl. Data Tables A7, 233 (1970)

^bJ. F. Ziegler, Helium Stopping Powers and Ranges in All Elemental Matter, Vol. 4 of The Stopping and Ranges of Ions In Matter Series (Pergamon Press, New York, 1977).

were fit to each set of tabulated stopping cross-section values, representing the values within 1% at any energy over the range $0.5 \text{ MeV} \leq E_\alpha \leq 10 \text{ MeV}$. These functional stopping cross sections are shown in Fig. 1. Coefficients of the polynomial functions are given in Table III. Stopping cross sections of the oxide fuels were formed from these component stopping cross-section functions using the Bragg-Kleeman relationship of Eq. (11) and component densities given in Table I.

Stopping cross-section values of UO_2 , ThO_2 , and $(\text{U}_{.8}\text{Pu}_{.2})\text{O}_2$ were computed over the range $2 \text{ MeV} \leq E_\alpha \leq 8 \text{ MeV}$ and compared in Table IV with values of ϵ converted from experimentally measured values of dE/dx reported by Nitzki and Matzke.⁷ The measured and calculated values of ϵ agree within 9% over this range, with calculated values generally lower than measured values.

B. (α, n) Cross Sections

The cross sections for the $^{17,18}\text{O}(\alpha, n)$ reactions have been reported over four limited ranges of E_α , although no single measurement extends over the entire range of our interest. Bair and Willard¹⁴ plotted their measured $^{18}\text{O}(\alpha, n)^{21}\text{Ne}$ cross-section values over the range $2.37 \text{ MeV} \leq E_\alpha \leq 5.15 \text{ MeV}$. Bair and Hass¹⁵ extended the range of these data down to 1.14 MeV and plotted the $^{17}\text{O}(\alpha, n)^{20}\text{Ne}$ cross section over the range $1.31 \text{ MeV} \leq E_\alpha \leq 5.31 \text{ MeV}$. Bair and del Campo¹⁶ later plotted the $\text{NAT}_0(\alpha, n)$ cross section over the range $3.1 \text{ MeV} \leq E_\alpha \leq 8 \text{ MeV}$ and, based on their measured $\text{NAT}_0(\alpha, n)$ neutron production by alpha particles in the range $4.62 \text{ MeV} \leq E_\alpha \leq 4.8 \text{ MeV}$, recommended that the $^{17,18}\text{O}(\alpha, n)$ cross sections reported in Refs. 14 and 15 be increased by 35%.

Differential cross sections $d\sigma(E)/d\Omega$ for $^{17,18}\text{O}(\alpha, n)$ reactions were measured at higher energies by Hansen et al.,¹⁷ who fit their measured angular distributions with Legendre polynomial expansions that they integrated to yield total $\sigma(\alpha, n)$ values. These values were plotted for the range $4.3 \text{ MeV} \leq E_\alpha \leq 12.3 \text{ MeV}$, and smooth curves were plotted approximating each set of data.

Except for cross-section values given by Hansen et al.¹⁷ at 9.8, 11.6, and 12.3 MeV, no data were available in other than graphic form--despite the best efforts of Bair,¹⁸ del Campo,¹⁹ and Hansen²⁰ to resurrect their numerical data. Data taken from the $^{17,18}\text{O}(\alpha, n)$ cross-section curves of Refs. 14 and 15 for the earlier HEDL work¹ were supplied to us.²¹ These data were thinned to 744 values of the $^{17}\text{O}(\alpha, n)$ cross section and 687 values of the $^{18}\text{O}(\alpha, n)$ cross section. Fourth-degree polynomial fits were made to data taken from the

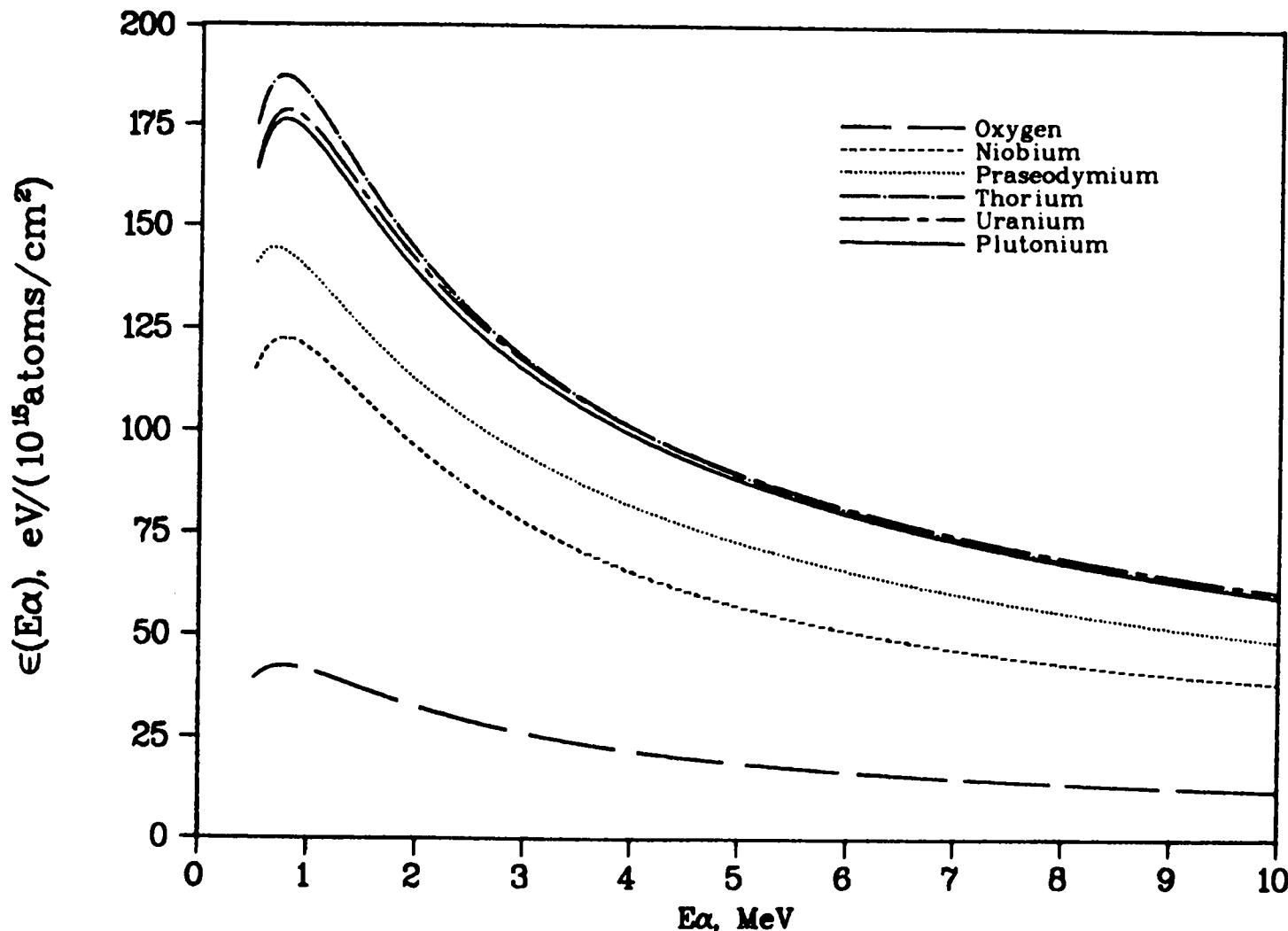


Fig. 1.
Stopping cross sections $\epsilon(E_\alpha)$ of O, Nb, Pr, Th, U, and Pu.

TABLE III
COEFFICIENTS OF POLYNOMIAL FITS TO STOPPING CROSS SECTIONS^a

Element	C ₀	C ₁	C ₂	C ₃	C ₄
O	3.7213	-0.168700	-0.300138	0.0700466	-0.00377296
Ni	4.7872	-0.156294	-0.278932	0.0533399	0.00186590
Pr	4.9321	-0.192312	-0.199561	0.0592391	-0.00940776
Th	5.2027	-0.195369	-0.278809	0.105037	-0.0163945
U	5.1648	-0.161478	-0.279242	0.099232	-0.0146254
Pu	5.1486	-0.171158	-0.272723	0.100975	-0.0160365

^a $\ln \epsilon = C_0 + C_1 \ln E + C_2 \ln^2 E + C_3 \ln^3 E + C_4 \ln^4 E$,
E is alpha-particle energy in MeV, $0.5 \leq E \text{ (MeV)} \leq 10.0$, and
 ϵ is stopping cross section in $\text{eV}/(10^{15} \text{ atoms/cm}^2)$.

TABLE IV
COMPARISON OF CALCULATED AND MEASURED
ALPHA STOPPING CROSS SECTIONS FOR OXIDE FUELS

$\epsilon(E)$ for ThO_2				$\epsilon(E)$ for UO_2				$\epsilon(E)$ for $(\text{U}_{.8}\text{Pu}_{.2})\text{O}_2$			
E MeV	From Table III and Eq. (11)			From Table III and Eq. (11)			From Table III and Eq. (11)				
	From a	% Dif		From a	% Dif		From a	% Dif		From a	% Dif
2	68.96	69.40	0.6	71.10	68.73	-3.3	72.17	68.55	-5.0		
3	59.38	56.27	-5.2	59.91	55.93	-6.6	60.48	55.84	-7.7		
4	52.13	48.67	-6.6	51.76	48.13	-7.0	52.05	48.01	-7.8		
5	46.46	42.43	-8.7	45.56	42.53	-6.6	45.69	42.43	-7.1		
6	41.91	38.20	-8.9	40.69	38.37	-5.7	40.71	38.27	-6.0		
7	38.16	34.87	-8.6	36.76	35.10	-4.5	36.71	35.00	-4.7		
8	35.03	32.23	-8.0	33.52	32.50	-3.0	33.42	32.41	-3.0		

^a Nitzki and Matzke, Phys. Rev. B8, 1894 (1973).

$\text{NAT}^0(\alpha, n)$ cross-section plot of Ref. 16 and to data taken from the $^{17, 18}\text{O}(\alpha, n)$ cross-section plots of Ref. 17. These five cross-section descriptions are shown in Fig. 2.

The $^{17, 18}\text{O}(\alpha, n)$ cross sections used in the present calculations were composed of the lower energy data of Refs. 14 and 15 increased by 35% as recommended in Ref. 16 and joined with the adjusted higher energy data of Ref. 17. This adjustment, amounting to a 9.2% reduction, was determined by normalizing the integral of the $\text{NAT}^0(\alpha, n)$ cross section formed from the functional fits to $^{17, 18}\text{O}(\alpha, n)$ cross sections of Ref. 17 to the integral of the $\text{NAT}^0(\alpha, n)$ cross section of Ref. 16 over the range $5.15 \text{ MeV} \leq E_\alpha \leq 8 \text{ MeV}$. The resulting adjusted cross sections are shown in Fig. 3. The adjusted $^{17}\text{O}(\alpha, n)$ cross section is given in Table V, and the adjusted $^{18}\text{O}(\alpha, n)$ cross section is given in Table VI; cross sections are defined there by interpolation points at low energies ($< 5 \text{ MeV}$) and by polynomial functions at higher energies.

C. Alpha-Decay Data

A total of 144 actinide nuclides produced in reactor fuel have been identified,²² using data of ENDF/B-V and Refs. 23-25. Of these, 89 decay at least partly by alpha decay. Each nuclide has some L different alpha-particle energies with $1 \leq L \leq 26$ for the data collection used. Alpha-particle energies in the data collection fall in the range $3.71 \text{ MeV} \leq E_\alpha \leq 8.78 \text{ MeV}$. TABLE VII lists the alpha-particle energies and intensities for each nuclide.

D. Spontaneous-Fission Data

Of the 144 actinide nuclides identified, 40 decay at least partly by spontaneous fission. Values of $\bar{v}_p(\text{SF})$, the major prompt contribution to $\bar{v}(\text{SF})$, are given by Manero and Konshin²⁶ for many of these. These values were used in Fig. 4 to estimate values of $\bar{v}_p(\text{SF})$ for nuclides without data.

Branching fractions F^{SF} , if not given in a data reference, were constructed from total and SF half-life values $T_{1/2}(\text{SF})$ using Eq. (19). Values of $T_{1/2}(\text{SF})$ given as limiting values were used and quoted without qualification. The values of $\bar{v}(\text{SF})$, F^{SF} , and $R(\text{SF})$ for each of the 40 nuclides are given in Table VIII.

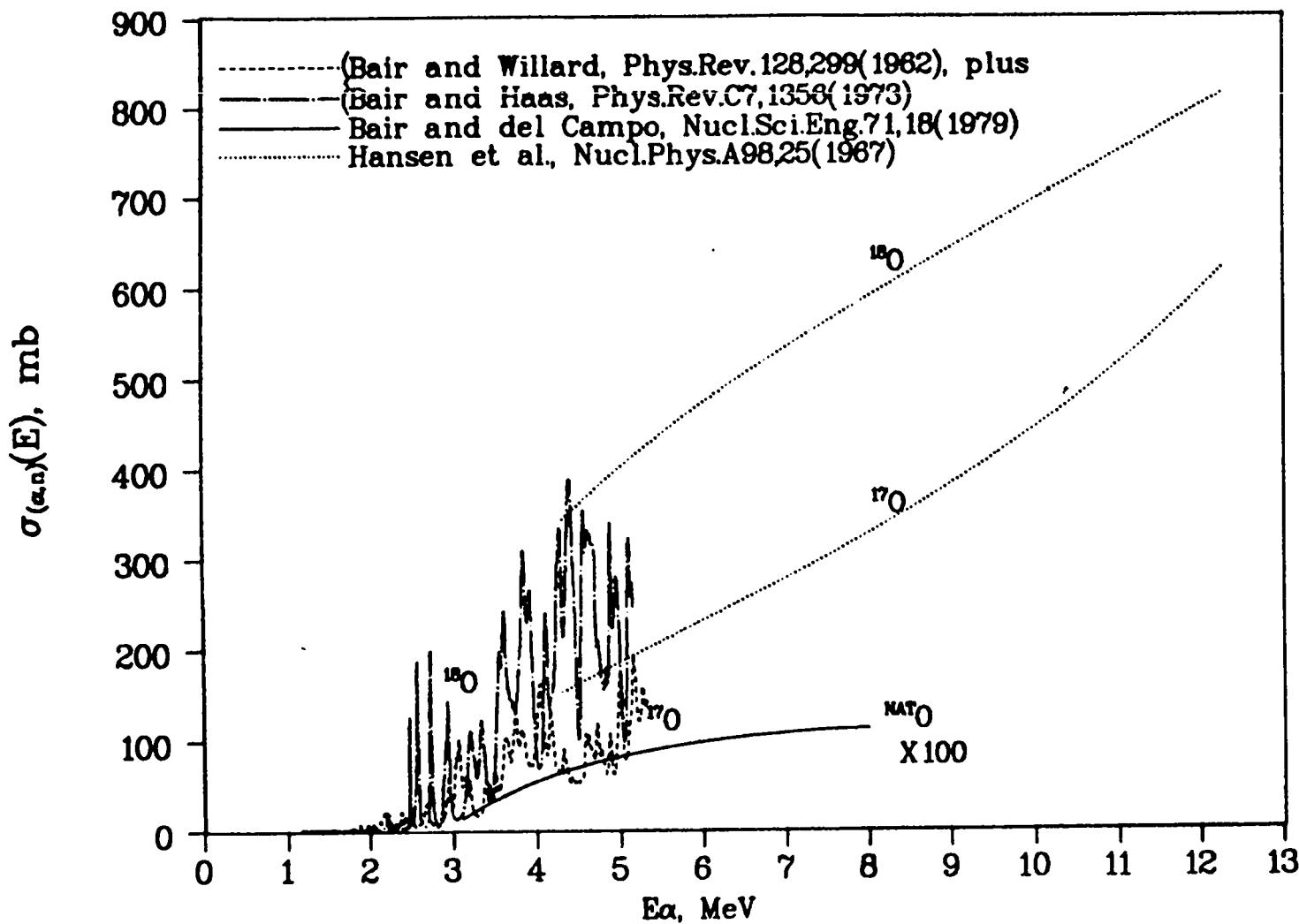


Fig. 2.
 ^{17}O , ^{18}O , and $^{\text{NAT}}\text{O}$ (α, n) cross-section data.

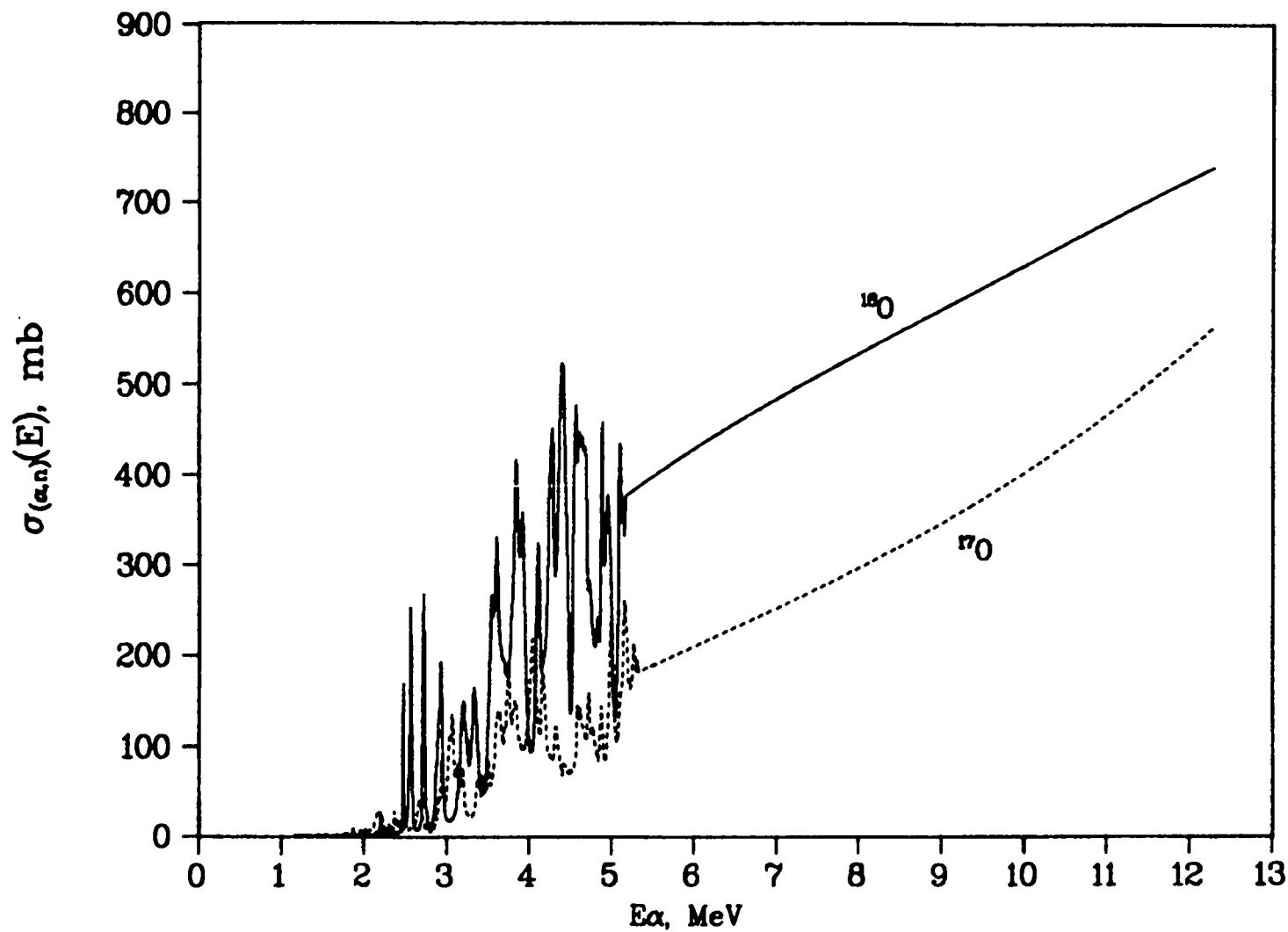


Fig. 3.
 ^{17}O and ^{18}O adjusted (α, n) cross sections.

TABLE V
ADJUSTED ^{17}O (α, n) CROSS SECTION

$E(\text{MeV})$	$CX(\text{mb})$																
1.3117	.03	1.5885	.04	1.8131	.94	2.1579	25.10	2.4845	13.61	2.8901	23.54	3.3322	27.73	3.7914	127.44	4.2865	85.87
1.3139	.03	1.5927	.12	1.8177	.78	2.1628	26.32	2.4901	18.44	2.8560	28.77	3.3083	28.88	3.7978	131.13	4.2933	90.84
1.3158	0.00	1.5968	.13	1.8221	.61	2.1677	26.23	2.4951	20.25	2.9016	45.71	3.3444	32.44	3.8041	140.13	4.3002	96.50
1.3178	.12	1.6007	.16	1.8265	.73	2.1726	25.92	2.5056	18.22	2.9072	44.74	3.3603	36.07	3.8105	144.96	4.3070	108.67
1.3195	.12	1.6051	.22	1.8311	.62	2.1775	28.09	2.5056	14.93	2.9128	41.46	3.3604	51.15	3.8171	150.16	4.3129	120.81
1.3214	.03	1.6054	.26	1.8356	.59	2.1824	26.23	2.5110	11.81	2.9241	42.80	3.3623	61.87	3.8234	150.04	4.3207	121.73
1.3234	.04	1.6135	.13	1.8400	2.07	2.1872	27.24	2.5161	9.09	2.9298	68.26	3.3686	64.62	3.8298	143.59	4.3276	113.21
1.3250	.07	1.6137	.26	1.8447	1.36	2.1921	26.45	2.5212	7.99	2.9354	83.75	3.3744	63.36	3.8361	145.39	4.3345	105.07
1.3272	0.00	1.6218	.49	1.8490	1.34	2.1970	25.34	2.5266	7.01	2.9564	71.05	3.3805	60.95	3.8427	141.48	4.3413	102.42
1.3286	.03	1.6232	.26	1.8523	1.46	2.2019	23.35	2.5317	6.98	2.9522	55.97	3.3867	58.85	3.8491	135.56	4.3481	133.54
1.3306	0.00	1.6246	.31	1.8580	1.30	2.2068	22.50	2.5371	6.52	2.9582	46.56	3.3925	56.66	3.8554	124.56	4.3550	94.84
1.3328	.03	1.6260	.39	1.8625	1.88	2.2117	25.25	2.5422	7.37	2.9638	39.27	3.3986	61.05	3.8623	118.17	4.3618	92.42
1.3345	.01	1.6301	.26	1.8671	1.32	2.2169	23.87	2.5474	6.84	2.9694	35.89	4.0407	67.80	3.8683	109.98	4.3686	86.66
1.3366	.07	1.6304	.53	1.8715	1.82	2.2217	19.08	2.5531	8.99	2.9753	35.84	4.1048	70.08	3.8749	105.29	4.3755	84.38
1.3385	.03	1.6343	.38	1.8761	.99	2.2265	16.21	2.5582	8.72	2.9809	35.05	4.169	69.48	3.8813	105.93	4.3823	78.26
1.3424	.03	1.6345	.43	1.8805	1.32	2.2314	14.46	2.5636	8.91	2.9846	65.65	4.230	65.08	3.8879	102.10	4.3891	74.88
1.3441	0.00	1.6385	.43	1.8882	1.16	2.2363	12.96	2.5689	8.06	2.9883	28.91	4.291	64.44	3.8942	102.10	4.3962	68.86
1.3461	.03	1.6388	.58	1.8899	1.44	2.2414	9.79	2.5741	8.72	2.9921	41.09	4.3525	62.23	3.9008	100.10	4.4033	68.67
1.3498	.01	1.6426	.35	1.8943	.93	2.2463	8.79	2.5792	8.30	2.9981	48.18	4.4113	59.79	3.9072	96.74	4.4104	76.87
1.3537	.04	1.6431	.32	1.8987	1.17	2.2512	8.40	2.5846	8.04	3.0037	54.88	4.4170	88.86	4.4170	80.39	4.4941	120.87
1.3576	.04	1.6467	.38	1.9036	.92	2.2564	5.94	2.5897	10.07	3.0093	65.73	4.4535	55.12	3.9201	100.08	4.4221	80.81
1.3616	.01	1.6475	.34	1.9079	1.03	2.2613	5.08	2.5951	10.79	3.0152	77.73	4.4596	52.94	3.9267	97.51	4.4530	79.44
1.3655	0.00	1.6509	.34	1.9125	1.42	2.2662	3.73	2.6004	12.30	3.0208	65.56	4.4658	51.58	3.9335	99.37	4.4380	77.36
1.3691	.01	1.6516	.45	1.9172	1.15	2.2707	2.86	2.6058	12.91	3.0264	107.08	4.4721	50.73	3.9399	99.35	4.4443	76.44
1.3730	.04	1.6560	.35	1.9217	.94	2.2713	2.56	2.6112	14.49	3.0322	110.11	4.4780	50.23	3.9462	96.92	4.4516	73.79
1.3770	.07	1.6558	.42	1.9265	1.28	2.2761	2.33	2.6167	15.15	3.0381	109.03	4.4841	52.61	3.9528	100.95	4.4585	71.23
1.3808	.07	1.6592	.45	1.9303	1.01	2.2807	2.27	2.6272	17.01	3.0441	107.07	4.4904	60.47	3.9594	104.52	4.4653	73.02
1.3845	.05	1.6602	.32	1.9356	.58	2.2810	2.59	2.6272	18.01	3.0494	107.51	4.4963	75.34	3.9660	113.58	4.4725	71.05
1.3884	.05	1.6633	.43	1.9402	2.07	2.2861	2.84	2.6323	18.48	3.0523	116.18	4.5026	85.10	3.9726	118.00	4.4763	70.31
1.3923	.04	1.6644	.53	1.9448	2.74	2.2908	8.82	2.6379	19.90	3.0609	127.87	4.5087	79.33	3.9792	125.25	4.4854	74.10
1.3952	.03	1.6678	.38	1.9496	4.31	2.2912	9.77	2.6433	22.55	3.0707	136.27	5.1548	70.31	3.9855	129.42	4.4933	73.22
1.4001	.05	1.6688	.42	1.9542	5.59	2.2957	12.60	2.6486	27.70	3.0729	129.71	5.2029	64.14	3.9921	137.71	4.5003	74.06
1.4041	.04	1.6720	.43	1.9585	7.13	2.2962	13.00	2.6540	32.39	3.0785	116.45	5.2573	59.20	3.9987	149.39	4.5069	71.29
1.4080	0.00	1.6732	.51	1.9634	8.50	2.3006	13.43	2.6594	35.53	3.0842	103.87	5.3134	57.54	3.0053	161.20	4.5140	77.10
1.4120	.01	1.6761	.38	1.9678	7.28	2.3011	13.35	2.6647	37.83	3.0901	87.45	5.3197	58.90	3.0119	178.00	4.5211	76.77
1.4198	.01	1.6774	.41	1.9727	7.03	2.3055	12.50	2.6701	40.45	3.0959	74.40	5.3458	60.16	3.0185	168.04	4.5279	77.37
1.4237	.03	1.6805	.45	1.9774	6.37	2.3062	12.01	2.6756	39.57	3.0959	74.40	5.3526	65.65	3.0251	202.23	4.5348	78.79
1.4276	.05	1.6818	.42	1.9818	6.21	2.3093	10.73	2.6810	36.45	3.1074	65.65	5.3680	69.19	3.0321	211.88	4.5421	78.62
1.4314	.07	1.6847	.36	1.9857	5.32	2.3124	11.11	2.6863	32.62	3.1135	70.04	5.3644	73.74	3.0384	217.76	4.5492	79.91
1.4354	.35	1.6859	.45	1.9913	5.02	2.3155	8.48	2.6917	28.62	3.1191	70.69	5.3707	76.45	3.0452	202.78	4.5558	84.36
1.4393	.17	1.6888	.73	1.9960	4.62	2.3162	9.00	2.6971	25.56	3.1251	77.18	5.3768	85.72	3.0515	202.66	4.5628	89.30
1.4432	.82	1.6903	.47	2.0006	4.32	2.3206	7.55	2.7027	23.38	3.1309	77.19	5.3832	96.11	3.0581	179.28	4.5697	103.94
1.4474	.90	1.6933	.42	2.0053	4.32	2.3214	8.18	2.7081	21.22	3.1368	80.70	5.3893	107.62	3.0650	166.52	4.5757	131.49
1.4513	.80	1.6944	.49	2.0100	6.58	2.3258	7.18	2.7134	19.14	3.1427	79.14	5.3959	119.69	3.0716	161.81	4.5841	144.41
1.4552	.74	1.6974	.92	2.0148	6.25	2.3284	1.44	2.7188	16.28	3.1485	77.18	5.4017	127.80	3.0782	159.50	4.5907	137.20
1.4592	.63	1.6988	.53	2.0194	7.07	2.3309	6.70	2.7244	14.80	3.1544	70.96	5.4074	126.66	3.0813	211.88	4.5421	78.62
1.4631	.54	1.7015	.53	2.0241	9.21	2.3359	6.56	2.7298	12.78	3.1635	70.64	5.4644	73.74	3.0848	217.76	4.5492	79.91
1.4670	.43	1.7032	.54	2.0287	8.29	2.3410	5.82	2.7352	10.92	3.1684	60.01	5.6205	135.01	3.0972	131.19	4.6119	142.13
1.4709	.30	1.7059	.49	2.0336	7.56	2.3461	5.33	2.7409	9.87	3.1721	54.85	5.6289	137.09	4.1045	122.88	4.6262	131.38
1.4751	.22	1.7076	.65	2.0382	6.56	2.3510	5.71	2.7462	8.61	3.1782	54.73	5.6330	136.92	4.1114	119.58	4.6262	131.38
1.4790	.22	1.7103	.58	2.0428	5.24	2.3613	7.76	2.7516	8.45	3.1838	65.45	5.6393	139.31	4.1180	125.45	4.6332	125.47
1.4829	.12	1.7118	.62	2.0477	5.24	2.3664	17.73	2.7572	8.07	3.1896	76.59	5.6445	135.31	4.1248	111.51	4.6403	123.11
1.4870	.38	1.7145	.63	2.0525	5.71	2.3715	25.93	2.7626	9.51	3.1955	74.64	5.6518	131.00	4.1314	116.06	4.6474	122.20
1.4909	.09	1.7162	.23	2.0574	5.64	2.3715	28.01	2.7680	7.01	2013	63.77	5.6581	119.88	4.1380	128.17	4.6545	118.35
1.4948	.07	1.7207	.69	2.0620	5.72	2.3765	26.00	2.7736	5.95	2072	51.54	5.6645	114.99	4.1451	152.50	4.6618	114.60
1.5029	.07	1.7251	.74	2.0668	6.13	2.3816	19.06	2.7792	5.53	2133	41.54	5.6708	110.74	4.1516	169.80	4.6686	114.43
1.5069	.09	1.7252	.74	2.0714	6.28	2.3865	14.51	2.7846	5.72	2192	35.30	5.6772	10				

TABLE VI
ADJUSTED ^{18}O (α, n) CROSS SECTION

$E(\text{MeV})$	$CX(\text{mb})$																
1.1448	0.00	1.5432	.68	1.8707	4.18	2.1957	27.20	2.5100	15.32	2.9240	168.47	3.3100	119.76	3.7240	177.86	4.1944	210.40
1.1482	.07	1.5474	.45	1.8751	1.48	2.2004	22.61	2.5140	17.46	2.9261	187.69	3.3180	135.78	3.7401	177.88	4.1984	215.74
1.1516	.05	1.5515	.58	1.8841	.43	2.2048	10.89	2.5180	22.80	2.9280	193.04	3.3234	156.07	3.7441	184.29	4.2064	230.70
1.1553	.07	1.5554	.31	1.8885	.34	2.2143	2.38	2.5261	29.21	2.9306	190.90	3.3287	164.62	3.7522	192.83	4.2104	246.73
1.1590	.05	1.5596	.23	1.8936	.31	2.2189	1.94	2.5301	31.77	2.9337	168.48	3.3341	163.57	3.7602	203.53	4.2225	280.91
1.1625	.05	1.5637	.23	1.8973	.23	2.2236	1.70	2.5381	61.26	2.9380	142.84	3.3461	148.62	3.7682	209.94	4.2295	316.16
1.1661	.01	1.5679	.22	1.8973	.23	2.2287	1.62	2.5421	69.82	2.9405	117.22	3.3502	140.04	3.7700	231.85	4.2426	356.50
1.1747	.00	1.5760	.19	1.9064	.22	2.2334	1.62	2.5462	79.43	2.9431	114.02	3.3542	128.34	3.7843	251.60	4.2547	404.81
1.1982	.13	1.5802	.16	1.9108	.18	2.2383	1.65	2.5542	124.29	2.9457	90.53	3.3622	119.81	3.7964	264.44	4.2687	412.29
1.2019	0.00	1.5843	.16	1.9152	.19	2.2430	1.70	2.5626	213.06	2.9482	74.56	3.3662	114.48	3.8004	279.38	4.2627	425.10
1.2055	0.00	1.5882	.13	1.9196	.16	2.2448	1.80	2.5689	191.58	2.9522	54.76	3.3703	103.80	3.8044	293.69	4.2740	450.75
1.2092	0.00	1.5926	.11	1.9242	.18	2.2525	1.86	2.5743	100.82	2.9642	36.10	3.3810	90.99	3.8205	322.11	4.2801	446.49
1.2640	0.00	1.5965	.16	1.9286	.19	2.2574	2.55	2.5823	57.04	2.9683	30.75	3.3863	81.39	3.8245	375.58	4.2855	424.06
1.2678	.01	1.6009	.11	1.9330	.18	2.2620	2.78	2.5864	35.69	2.9723	26.94	3.3904	69.65	3.8326	399.02	4.2908	377.08
1.2826	.01	1.6051	.09	1.9377	.19	2.2669	2.71	2.5864	34.93	2.9723	24.35	3.3971	57.91	3.8366	415.04	4.2949	337.58
1.2863	0.00	1.6093	.09	1.9423	.23	2.2715	2.19	2.5904	25.02	2.9763	24.35	3.4038	51.51	3.8406	416.16	4.3029	303.41
1.3013	0.00	1.6135	.11	1.9467	.57	2.2764	1.97	2.5944	20.75	2.9843	22.23	3.4105	45.12	3.8486	400.10	4.3109	289.81
1.3052	.01	1.6176	.09	1.9512	.39	2.2810	2.02	2.5984	15.42	2.9924	20.10	3.4185	47.45	3.8527	373.31	4.3230	301.26
1.3089	.01	1.6215	.08	1.9556	.43	2.2850	1.98	2.6018	11.28	2.9964	19.05	3.4226	46.40	3.8627	332.13	4.3310	318.20
1.3126	.03	1.6240	.08	1.9602	.28	2.2955	2.20	2.6132	9.02	3.0162	16.00	3.4262	57.55	3.8687	332.82	4.3391	349.37
1.3164	.16	1.6381	.07	1.9646	.57	2.3020	2.34	2.6182	1.97	3.0283	16.94	3.4346	64.35	3.8728	314.70	4.3431	364.32
1.3201	.43	1.6423	.08	1.9693	.30	2.3032	2.47	2.6266	6.90	3.0328	16.94	3.4386	49.41	3.8808	310.45	4.3512	394.23
1.3238	.18	1.6464	.07	1.9739	.30	2.3051	2.47	2.6386	6.91	3.0446	16.94	3.4453	47.29	3.8848	308.31	4.3592	433.75
1.3277	.15	1.6548	.07	1.9783	.27	2.3097	2.73	2.6386	6.91	3.0446	16.94	3.4453	49.42	3.8929	320.06	4.3672	452.68
1.3316	.07	1.6590	.08	1.9829	.26	2.3148	3.05	2.6467	6.93	3.0487	18.04	3.4520	50.77	3.8969	336.08	4.3753	485.03
1.3353	.01	1.6631	.08	1.9876	.30	2.3195	2.20	2.6547	6.93	3.0567	19.10	3.4587	54.77	3.9009	49.10	4.3867	485.77
1.3390	0.00	1.6675	.07	1.9921	.34	2.3244	6.70	2.6681	8.01	3.0688	20.18	3.4667	60.12	3.9130	357.26	4.3973	529.57
1.3466	0.00	1.6841	.07	1.9967	.35	2.3290	9.46	2.6781	10.15	3.0728	21.26	3.4708	75.07	3.9170	357.47	4.3993	522.42
1.3505	0.01	1.6885	.09	2.0013	.34	2.3344	10.98	2.6873	11.22	3.0848	22.34	3.4748	75.07	3.9210	357.47	4.4033	517.09
1.3544	0.00	1.6930	.05	2.0055	.41	2.3388	9.74	2.6888	14.43	3.0929	23.42	3.4868	88.85	3.9210	331.81	4.4073	504.28
1.3583	0.00	1.6972	.07	2.0104	.43	2.3436	7.49	2.6889	15.51	3.0969	25.56	3.4909	99.66	3.9331	308.35	4.4113	487.20
1.3619	.01	1.7013	.07	2.0148	.51	2.3485	6.10	2.6949	25.12	3.1009	27.70	3.4989	114.62	3.9371	273.12	4.4113	487.20
1.3659	0.00	1.7057	.08	2.0284	.51	2.3533	4.83	2.6989	36.88	3.1049	29.84	3.5029	129.57	3.9491	235.76	4.4195	459.45
1.3696	.01	1.7096	.09	2.0331	.49	2.3581	4.23	2.7029	64.65	3.1090	30.91	3.5080	152.01	3.9532	208.01	4.4275	429.56
1.3811	.01	1.7140	.07	2.0377	.58	2.3630	4.06	2.7070	141.53	3.1130	31.91	3.5142	166.95	3.9612	175.97	4.4315	398.56
1.3850	0.00	1.7228	.07	2.0424	.62	2.3679	3.89	2.7119	224.15	3.1170	36.26	3.5198	173.48	3.9652	161.03	4.4396	376.18
1.4007	0.00	1.7270	.09	2.0470	.65	2.3728	3.58	2.7271	267.57	3.1210	41.59	3.5254	196.86	3.9733	141.82	4.4436	362.30
1.4046	.01	1.7311	.08	2.0515	.68	2.3777	2.25	2.7311	186.55	3.1291	46.95	3.5311	210.76	3.9813	129.01	4.4516	330.26
1.4083	0.00	1.7353	.08	2.0561	.68	2.3826	0.09	2.7351	128.62	3.1331	49.09	3.5331	220.37	3.9853	112.99	4.4597	305.72
1.4122	0.00	1.7398	.13	2.0608	.69	2.3877	3.10	2.7391	78.57	3.1411	58.71	3.5351	237.45	3.9893	105.53	4.4637	272.62
1.4161	.01	1.7442	.07	2.0654	.73	2.3926	3.04	2.7431	56.15	3.1492	58.32	3.5471	266.30	3.9974	101.26	4.4717	240.60
1.4239	.01	1.7483	.08	2.0700	.74	2.3934	.52	2.7458	43.33	3.1572	76.88	3.5512	246.70	4.0054	95.93	4.4758	200.02
1.4281	.16	1.7527	.09	2.0747	.80	2.4014	.46	2.7485	38.86	3.1612	84.36	3.5592	242.82	4.0175	93.80	4.4838	169.06
1.4320	.01	1.7571	.08	2.0791	.96	2.4095	.47	2.7512	30.52	3.1652	96.11	3.5632	242.82	4.0175	93.81	4.4918	145.57
1.4360	.03	1.7612	.09	2.0840	.84	2.4135	.47	2.7579	27.34	3.1693	110.00	3.5672	246.04	4.0255	98.89	4.4959	135.57
1.4399	.15	1.7654	.08	2.0884	.94	2.4215	.55	2.7645	21.99	3.1773	118.54	3.5753	255.65	4.0336	95.96	5.0064	187.77
1.4435	.26	1.7700	.09	2.0930	1.04	2.4376	.67	2.7713	14.53	3.1853	128.16	3.5824	265.57	4.0456	101.32	5.1560	168.02
1.4477	.24	1.7747	.09	2.0976	1.07	2.4416	.67	2.7753	10.26	3.1893	142.05	3.5914	265.57	4.0496	113.06	5.2020	238.50
1.4516	.24	1.7782	.11	2.1023	.15	2.4497	.78	2.7833	10.27	3.1974	147.39	3.5994	312.28	4.0577	121.62	4.5280	317.53
1.4555	.19	1.7870	.11	2.1069	.38	2.4517	12.06	2.7914	10.29	3.2014	149.54	3.6034	330.44	4.0657	142.98	4.5361	382.68
1.4594	.08	1.7914	.09	2.1117	.17	2.4537	14.19	2.7994	11.35	3.2054	147.41	3.6074	331.76	4.0738	172.89	4.5441	419.00
1.4633	.05	1.7958	.08	2.1160	.62	2.4557	16.34	2.8075	12.43	3.2095	142.07	3.6155	302.68	4.0818	219.89	4.5522	452.11
1.4790	.05	1.8004	.09	2.1207	.97	2.4577	18.47	2.8155	13.51	3.2148	130.34	3.6195	270.65	4.0858	246.59	4.5602	476.68
1.4832	.08	1.8046	.11	2.1256	.21	2.4617	.95	2.8195	15.65	3.2202	121.78	3.6275	246.11	4.0939	282.91	4.5682	435.04
1.4871	.08	1.8087	.15	2.1300	.52	2.4648	.86	2.8213	16.73	3.2255	120.83	3.6316	233.29	4.0979	312.81	4.5763	413.69
1.4910	.11	1.8131	.15	2.1348	.01	2.4678	9.30	2.8235	19.94	3.2336	106.83	3.6396	222.51	4.1059	323.49	4.5883	428.52
1.4952	.16	1.8177	.22	2.1392	.51	2.4708	108.18	2.8436	23.15	3.2376	109.00	3.6436	213.02	4.1140	309.62	4.5950	434.54
1.4991	.19	1.8220	.18	2.14													

TABLE VII
ALPHA DECAY SPECTRA OF ACTINIDE NUCLIDES

82-PB-210	1 ALPHA REF B E, MEV. DK FRACTION	83-BI-214	6 ALPHAS, REF B E, MEV. DK FRACTION	86-RN-218	2 ALPHAS, REF B E, MEV. DK FRACTION	88-RA-223	12 ALPHAS, KEF B E, MEV. DK FRACTION	89-AC-227	8 ALPHAS, REF B E, MEV. DK FRACTION	90-TH-230	7 ALPHAS, KEF A E, MEV. DK FRACTION
3.7198	1.70000E-08	4.9420	5.25210E-07	6.5349	1.40091E-03	5.2839	1.00000E-03	5.715	4.30252E-05	4.2840	5.06746E-08
		5.0234	4.41176E-07	7.1331	9.98099E-01	5.2885	1.30000E-03	5.7701	1.434308E-04	4.2780	8.01194E-08
83-BI-210	2 ALPHAS, REF B E, MEV. DK FRACTION	5.1824	1.26050E-06	85-AT-219	1 ALPHA REF B E, MEV. DK FRACTION	5.3399	1.00000E-03	5.7957	1.12421E-04	4.3720	1.00149E-05
		5.2638	1.21849E-05	5.4508	1.13235E-04	5.3660	1.10000E-03	5.8556	5.13527E-04	4.4380	3.00448E-04
		5.5121	8.23529E-05	5.5125	1.00000E-02	5.3437	2.30000E-02	5.8733	8.46626E-04	4.4800	1.20179E-03
4.6879	7.80000E-07	4.6861	5.20000E-07	6.2733	9.70000E-01	5.5396	9.10000E-02	5.9008	1.52670E-05	4.6210	2.34349E-01
						5.6073	2.40000E-01	5.9416	1.55164E-03	4.6875	7.544136E-01
84-PO-210	2 ALPHAS, REF B E, MEV. DK FRACTION	84-PO-214	2 ALPHAS, REF B E, MEV. DK FRACTION	86-RN-219	4 ALPHAS, REF B E, MEV. DK FRACTION	5.7161	5.28900E-01	4.9558	6.52318E-03	91-PA-230	18 ALPHAS, KEF B E, MEV. DK FRACTION
		6.9025	1.00000E-04	5.8577	3.20000E-03	5.7874	9.10000E-02			4.7653	6.40446E-08
4.5168	1.07000E-05	7.6873	9.99900E-01	5.8718	8.50000E-03	5.8718	8.50000E-03			4.7987	9.60672E-09
5.3046	9.99989E-01			6.4250	7.49101E-02	5.0340	3.09945E-05	5.5862	1.81892E-03	4.9343	1.28090E-07
		84-PO-215	3 ALPHAS, REF B E, MEV. DK FRACTION	6.5310	1.19856E-03	5.6021	1.71787E-03	5.6136	2.22312E-03	4.9726	2.24157E-07
		6.5532	1.14862E-01	6.8194	8.09029E-01	5.6666	2.12207E-03	5.6000	1.28090E-07	5.0836	2.24157E-07
83-BI-211	2 ALPHAS, REF B E, MEV. DK FRACTION	6.9497	2.20000E-04	86-RN-220	2 ALPHAS, REF A E, MEV. DK FRACTION	5.0740	3.09945E-05	5.7017	3.63783E-02	5.1190	1.92134E-07
		6.9559	3.40000E-04	6.9559	3.40000E-04	5.1610	7.28872E-05	5.7097	8.28618E-02	5.1534	1.28090E-07
		7.3865	9.99440E-01	5.4490	4.88914E-02	5.1142	4.95150E-02	5.1859	1.60112E-07	5.2173	1.60112E-07
6.2790	1.59562E-01			5.6856	9.50833E-01	5.7572	2.02102E-01	5.7630	2.32417E-03	5.2683	1.12078E-06
6.6233	8.37698E-01	85-AT-215	2 ALPHAS, REF B E, MEV. DK FRACTION	6.2883	9.99300E-01	5.7952	3.13258E-03	5.8076	3.13258E-03	5.2762	9.60672E-07
		6.6285	5.00000E-04	87-FR-221	4 ALPHAS, REF B E, MEV. DK FRACTION	5.8667	2.42522E-02	5.8867	2.42522E-02	5.2880	9.60672E-07
6.5694	5.40000E-03	8.0258	9.99500E-01	5.8718	8.50000E-03	5.9103	1.71787E-03	5.3008	5.44381E-06	5.3126	1.62421E-06
6.8914	5.50000E-03			5.8718	8.50000E-03	5.9166	7.88197E-03	5.3263	5.76403E-06	5.3401	4.80338E-06
7.4502	9.89100E-01	84-PO-216	2 ALPHAS, REF A E, MEV. DK FRACTION	6.0752	1.36426E-03	5.9599	3.03193E-02	5.3450	7.36516E-06		
		6.1275	1.37335E-01	6.1275	1.37335E-01	5.9779	2.32417E-01				
		6.2434	1.11236E-02	6.2809	1.20084E-02	6.0089	2.93048E-02	6.0383	2.42522E-01	92-U-230	4 ALPHAS, KEF B E, MEV. DK FRACTION
		6.3411	8.49477E-01	6.3411	8.49477E-01	5.6824	1.25088E-02			5.6622	2.30023E-03
83-BI-212	8 ALPHAS, REF B E, MEV. DK FRACTION	5.9850	2.10000E-05	86-RN-222	2 ALPHAS, REF B E, MEV. DK FRACTION	5.7235	3.40238E-02	5.6661	3.60030E-03	5.8176	3.19032E-01
		6.7705	9.99979E-01	5.9850	2.10000E-05	5.7320	1.01071E-01	5.1770	1.80018E-03	5.8886	6.75068E-01
5.3024	3.95857E-07			5.9870	7.80117E-04	5.8043	3.00210E-03	5.2110	3.60036E-03	91-PA-231	19 ALPHAS, KEF A E, MEV. DK FRACTION
3.3456	3.59870E-06	5.9898	5.39805E-05	5.9898	9.99220E-01	5.8299	5.06855E-01	5.3405	2.67027E-01	5.4233	7.27073E-01
4.4889	3.59805E-05			87-FR-222	1 ALPHA, REF B E, MEV. DK FRACTION	5.6377	4.35305E-02	5.6824	4.25088E-02		
6.0698	3.59857E-03			5.7092	1.00000E-03	5.7235	3.40238E-02	5.6661	3.60030E-03		
6.6258	3.59805E-04			5.7092	1.00000E-03	5.7320	1.01071E-01	5.6661	3.60030E-03		
7.7688	6.00983E-03	6.4846	4.00100E-04	5.7092	1.00000E-03	5.7320	1.01071E-01	5.1380	5.00050L-04	91-PA-231	19 ALPHAS, KEF A E, MEV. DK FRACTION
6.0511	2.51249E-01	6.6113	1.00025E-04	5.7092	1.00000E-03	5.7320	1.01071E-01	5.1770	1.80018E-03	5.5090	3.02822E-05
6.0902	9.76847E-02	6.8134	2.50063E-04	5.7092	1.00000E-03	5.7320	1.01071E-01	5.6660	8.07526E-05	5.6330	1.00941E-03
		7.0677	9.99250E-01	5.7092	1.00000E-03	5.7320	1.01071E-01	5.6330	1.00941E-03	5.6440	1.00941E-03
84-PO-212	1 ALPHA, REF B E, MEV. DK FRACTION	86-RN-217	1 ALPHA, REF B E, MEV. DK FRACTION	5.7092	1.00000E-03	5.7092	1.00000E-03	5.7320	1.01071E-01	5.6820	1.51411E-02
		8.7846	1.00000E+00	5.7092	1.00000E-03	5.7092	1.00000E-03	5.7320	1.01071E-01	5.7130	1.00941E-02
83-BI-213	2 ALPHAS, REF B E, MEV. DK FRACTION	84-PO-218	2 ALPHAS, REF B E, MEV. DK FRACTION	6.2373	3.10000E-02	5.4400	6.00000E-05	5.8460	5.67447E-01	5.7360	8.47902E-02
		5.5508	1.62963E-03	6.5557	9.69000E-01	5.4400	6.00000E-05	5.8622	1.81745E-03	5.7980	4.03763E-04
5.8687	2.03704E-02	5.1810	1.10000E-05	5.3330	4.00000E-05	5.4400	6.00000E-05	5.9017	1.09047E-01	5.8540	1.41317E-02
		6.0027	9.99989E-01	5.3330	4.00000E-05	5.4400	6.00000E-05	5.9020	2.01882E-05	5.9340	3.02822E-02
84-PO-213	2 ALPHAS, REF B E, MEV. DK FRACTION	85-AT-218	3 ALPHAS, REF B E, MEV. DK FRACTION	6.6625	6.40000E-02	5.0258	2.00682E-03	5.9795	3.23102E-02	5.9517	2.30145E-01
		6.7045	9.00000E-01	6.0414	1.90648E-03	6.0997	1.27433E-02	5.0263	2.52326E-03	5.9760	4.03763E-03
7.6123	3.00000E-05	6.3757	9.99970E-01	6.2284	2.30785E-01	6.2284	2.30785E-01	5.0492	5.25040E-02	5.9860	1.41317E-02
		6.7567	3.60000E-02	6.3375	7.52559E-01	6.3375	7.52559E-01	5.0534	1.61551E-02	5.0141	2.56390E-01
						5.0783	1.00969E-04	5.0297	2.01882E-01	5.0320	2.52352E-02
								5.0590	1.11035E-01		

TABLE VII (cont.)

92-U-231	92-U-235	93-NP-237	94-PU-239	95-AM-241	96-CM-242
1 ALPHA, REF B E,MEV. DK FRACTION	12 ALPHAS, REF A E,MEV. DK FRACTION	14 ALPHAS, REF A E,MEV. DK FRACTION	21 ALPHAS, REF A E,MEV. DK FRACTION	21 ALPHAS, REF A E,MEV. DK FRACTION	8 ALPHAS, REF A E,MEV. DK FRACTION
5.4539 5.50000E-05	4.1540 9.00000E-03	4.5140 3.98843E-04	4.3990 2.50024E-07	4.8000 9.00775E-07	5.1460 4.99798E-08
5.4539 5.50000E-05	4.2170 5.70000E-02	4.5810 3.98843E-03	4.6100 8.00078E-07	4.8340 7.000603E-06	5.1890 2.49898E-07
90-TH-232	3 ALPHAS, REF A E,MEV. DK FRACTION	4.2270 9.00000E-03	4.5980 3.39017E-03	4.6300 7.00068E-06	5.5140 2.49898E-06
3.8300 1.99601E-03	4.2710 4.70000E-02	4.6391 6.16213E-02	4.6890 5.00048E-06	5.0040 1.00066E-06	5.6090 1.99918E-07
3.8300 1.99601E-03	4.3240 4.70000E-02	4.6640 3.31040E-02	4.7360 4.50044E-05	5.0680 1.40121E-06	5.8170 4.59812E-05
3.8300 1.99601E-03	4.3680 1.70000E-01	4.6940 7.78512E-03	4.7490 6.00058E-06	5.0890 4.00345E-06	5.9720 3.59853E-04
3.8300 1.99601E-03	4.3980 5.60000E-01	4.7120 1.12573E-02	4.7690 8.00078E-06	5.1140 8.00345E-06	6.0696 2.59d44E-01
3.9530 2.29541E-01	4.4160 2.10000E-02	4.7659 7.97687E-02	4.7950 7.00068E-06	5.1560 7.000603E-06	6.1129 7.39897E-01
4.0120 7.68463E-01	4.4390 7.00000E-03	4.7701 2.49277E-01	4.8280 2.40023E-05	5.1780 3.00258E-06	
92-U-232	7 ALPHAS, REF A E,MEV. DK FRACTION	4.5560 4.50000E-02	4.8030 2.99133E-02	4.9110 2.00019E-06	5.1820 9.00775E-06
4.5090 2.39800E-07	4.5980 5.40000E-02	4.8170 2.49277E-02	4.9350 3.00029E-05	5.1940 6.00517E-06	
4.9291 2.09825E-06	93-NP-235	11 ALPHAS, REF B E,MEV. DK FRACTION	4.8660 2.99133E-03	4.9620 3.00029E-05	5.2230 1.30112E-05
4.9460 1.69859E-06	4.8042 1.60160E-08	4.8730 2.59248E-02	4.9870 7.00068E-05	5.2440 4.02070E-05	
4.9973 2.89775E-05	4.8599 1.12112E-07	5.0080 8.00078E-05	5.2790 5.00431E-06	5.2790 5.00431E-06	
5.1373 2.79767E-03	4.9229 1.84184E-06	5.0280 5.00048E-05	5.3220 1.50124E-04	4.6950 1.60298E-05	
5.2633 3.11740E-01	4.9365 9.60961E-08	5.0480 5.00048E-05	5.3690 1.33115E-02	4.8190 8.51572E-07	
5.3203 6.85429E-01	4.9933 9.60961E-07	5.0540 2.10020E-04	5.4170 1.00086E-04	4.9300 1.80333E-06	
92-U-233	25 ALPHAS, REF A E,MEV. DK FRACTION	5.0025 8.00801E-08	5.0750 3.20031E-04	5.4430 2.82110E-01	4.9460 3.40629E-06
5.0033 8.84384E-06	5.0203 8.48849E-06	5.0800 8.00078E-05	5.4857 8.52734E-01	5.0080 1.60296E-05	
5.0455 2.88828E-07	5.0455 2.88828E-07	5.0860 7.00068E-05	5.5130 2.00112E-03	5.0320 2.20407E-05	
4.3070 9.01370E-06	5.0940 3.20320E-08	5.1030 2.40240E-07	5.5443 3.50302E-03	5.0860 4.00740E-05	
4.4060 8.01218E-06	4.4570 2.80826E-05	4.4570 1.40213E-05	5.1130 5.40999E-05	5.1810 1.10203E-02	
4.4570 2.80826E-05	94-PU-235	1 ALPHA, REF B E,MEV. DK FRACTION	5.1554 7.33071E-01	5.2335 1.10203E-01	
4.4650 3.00857E-05	5.0530 1.00152E-05	5.1554 7.33071E-01	5.2754 8.75616E-01	5.2754 8.75616E-01	
4.4830 1.40213E-05	5.1030 2.40240E-07	5.1683 7.34051E-01	5.3210 1.20222E-03	5.3210 1.20222E-03	
5.0530 1.00152E-05	94-PU-238	3 ALPHAS, REF A E,MEV. DK FRACTION	5.6870 2.20044E-05	5.3492 1.80333E-03	5.3492 1.80333E-03
5.0704 1.20183E-04	5.1300 1.80271E-04	5.1790 8.00106E-06	5.6870 2.20044E-05		
5.1300 1.80271E-04	5.1556 1.30000E-04	5.1800 2.29472E-03	5.7190 8.00106E-06	5.2230 1.10203E-01	
5.1580 4.00609E-05	5.1827 1.12112E-07	5.1810 2.29472E-01	5.7514 8.00106E-06	5.2754 8.75616E-01	
5.1827 1.12112E-07	5.1960 7.68233E-01	5.1820 2.29472E-01	5.7850 7.000140E-06	5.3150 9.90940E-06	
5.1960 7.68233E-01	92-U-236	11 ALPHAS, REF A E,MEV. DK FRACTION	5.8630 1.40028E-05	5.3220 2.97282E-05	5.3220 2.97282E-05
5.2020 7.01066E-05	3 ALPHAS, REF A E,MEV. DK FRACTION	5.1234 2.65018E-01	5.8843 1.18024E-03	5.3310 2.97282E-05	
5.2110 6.00913E-05	5.0680 1.60104E-08	5.1234 2.65018E-01	5.9180 1.20024E-05	5.2260 3.88467E-06	
5.2150 4.00609E-05	5.0680 1.60104E-08	5.1300 2.33630E-08	5.9260 1.81036E-03	5.0660 1.46641E-05	
5.2650 2.80426E-05	4.5670 1.10071E-09	5.1370 2.371932E-07	5.9386 6.89138E-03	5.3150 9.90940E-06	
5.2720 2.30350E-05	4.5880 1.60104E-08	5.1370 2.371932E-07	5.9780 2.80056E-05	5.3220 2.97282E-05	
5.2900 7.01066E-05	4.6620 3.00195E-08	5.1370 2.371932E-07	6.0360 1.20024E-05	5.3310 2.97282E-05	
5.6110 6.00913E-05	4.6680 3.00195E-08	5.1370 2.371932E-07	6.0820 1.50030E-05	5.5220 1.98186E-05	
5.6340 1.00152E-04	4.7310 2.59326E-03	5.1370 2.371932E-07	5.8843 1.18024E-03	5.5310 5.94564E-05	
5.6410 3.00457E-05	4.4450 2.59326E-01	5.1400 7.00455E-08	5.9180 1.20024E-05	5.5360 1.98186E-05	
5.6560 4.00609E-05	4.4940 7.38081E-01	5.1400 7.00455E-08	5.9260 1.81036E-03	5.5670 6.93658E-05	
5.6640 4.20639E-04	94-PU-236	5.0110 7.00455E-08	5.9780 2.80056E-05	5.5740 6.93658E-05	
5.6810 1.00152E-04	6 ALPHAS, REF A E,MEV. DK FRACTION	5.2080 5.00325E-05	5.985 1.30001E-05	5.5810 8.91846E-05	
5.7010 6.00913E-04	5.0455 2.87186E-01	5.2080 5.00325E-05	5.986 1.24002E-01	5.5860 1.98186E-04	
5.7290 1.61245E-02	5.0880 6.00103E-06	5.2080 5.00325E-05	5.9909 1.339908E-04	5.5920 9.90940E-05	
5.7540 1.63248E-03	5.2140 2.70046E-06	5.2080 5.00325E-05	5.992 2.24002E-01	5.6080 9.90940E-05	
5.7825 1.32201E-01	5.4520 2.00034E-05	5.2080 5.00325E-05	5.996 7.75005E-01	5.6110 3.96376E-04	
5.7960 2.80426E-03	5.6150 1.80031E-03	5.2080 5.00325E-05	5.998 6.24002E-01	5.6210 5.94564E-04	
4.8242 8.45285E-01	5.7210 3.09053E-01	5.2080 5.00325E-05	6.2469 2.88809E-01	5.6380 1.38732E-03	
5.7680 6.89118E-01	5.7680 6.89118E-01	5.2080 5.00325E-05	6.2908 7.10531E-01	5.6450 2.37262E-04	
92-U-234	5 ALPHAS, REF A E,MEV. DK FRACTION	94-PU-241	94-PU-242	95-AM-242M	96-CM-242
5.1200 3.29013E-07	4.5140 3.98843E-04	11 ALPHAS, REF A E,MEV. DK FRACTION	11 ALPHAS, REF A E,MEV. DK FRACTION	7 ALPHAS, REF A E,MEV. DK FRACTION	8 ALPHAS, REF A E,MEV. DK FRACTION
5.2740 4.46054E-07	4.5810 3.98843E-03	4.5140 3.98843E-04	4.5920 7.35662E-09	5.6810 1.96186E-03	5.1460 4.99798E-08
5.6030 2.99102E-03	4.6640 3.31040E-02	4.5810 3.98843E-03	5.1400 2.90537E-04	5.6850 1.58559E-02	5.1890 2.49898E-07
5.7228 2.74177E-01	4.7120 1.12573E-02	4.6890 5.00048E-06	5.2050 4.27401E-03	5.7415 1.13956E-01	5.5140 2.49898E-06
5.7730 7.22831E-01	4.7690 7.97687E-02	4.7360 4.50044E-05	5.3130 3.84181E-05	5.7847 7.25368E-01	5.6090 9.40940E-04
		4.7490 6.00058E-06	5.3650 7.20339E-05		5.8750 6.93658E-03
		4.7690 8.00078E-06	5.4080 5.70271E-05		5.9060 9.40940E-04
		4.8965 2.04024E-05	5.4080 5.70271E-05		5.9920 5.64836E-02
		4.9720 3.18787E-07	5.4080 5.70271E-05		6.0090 1.09003E-02
		5.6990 1.00540E-07	5.4080 5.70271E-05		6.0560 4.65742E-02
		5.0420 2.50125E-07	5.4080 5.70271E-05		6.0670 1.48641E-02
		5.0540 8.58272E-08			

TABLE VII (cont.)

94-PU-244	97-BK-249	98-CF-252	99-ES-254
2 ALPHAS, REF A E, MEV. DK FRACTION	7 ALPHAS, REF A E, MEV. DK FRACTION	5 ALPHAS, REF A E, MEV. DK FRACTION	8 ALPHAS, REF B E, MEV. DK FRACTION
4.5460 1.93757E-01	5.0450 1.45291E-06	5.6160 5.80623E-07	6.2669 2.21953E-03
4.5890 8.04993E-01	5.1150 3.92285E-07	5.8253 1.93541E-05	6.2758 1.61421E-03
96-CM-244	5.2530 1.45291E-06	5.9766 2.32249E-03	6.3467 7.46570E-03
8 ALPHAS, REF A E, MEV. DK FRACTION	5.3610 3.77566E-07	6.0757 1.51930E-01	6.3595 2.92575E-02
5.3899 3.61335E-06	5.4168 1.00541E-05	6.1183 8.14807E-01	6.3841 1.31164E-03
5.4373 9.73447E-07	5.4373 9.73447E-07	6.1183 8.14807E-01	6.4155 1.11509E-02
9.9200 8.99767E-07	98-CF-249	98-CF-253	6.4263 9.38257E-01
9.9600 1.99948E-06	16 ALPHAS, REF A E, MEV. DK FRACTION	2 ALPHAS, REF A E, MEV. DK FRACTION	6.4780 2.12397E-03
5.2150 1.19969E-06	5.3510 1.99410E-05	5.9210 1.64300E-04	99-ES-254
5.3130 3.99896E-07	5.4310 9.37049E-05	5.9790 2.93570E-03	7 ALPHAS, REF B E, MEV. DK FRACTION
5.5130 3.49909E-05	5.5020 9.38701E-04	99-ES-253	6.2472 7.37056E-05
6.6400 2.19943E-04	5.5590 1.09675E-03	28 ALPHAS, REF A E, MEV. DK FRACTION	6.2797 2.78071E-04
7.7628 2.35939E-01	5.6230 1.99410E-04	5.9100 2.70015E-07	6.3045 2.51269E-03
5.8050 7.63602E-01	5.6940 2.49262E-03	5.9350 1.00222E-07	6.3405 6.03046E-06
96-CM-245	5.7597 7.8583E-02	5.7300 8.00043E-07	6.3561 4.69030E-05
6 ALPHAS, REF A E, MEV. DK FRACTION	5.7840 2.49262E-03	5.9100 2.70015E-07	6.3791 1.94315E-04
5.2346 3.20256E-03	5.8120 2.35562E-01	5.9350 1.00222E-07	6.5130 1.34010E-04
5.3038 4.97398E-02	5.8945 1.39587E-02	6.0190 1.80010E-06	100-FM-254
5.3620 9.32846E-01	5.9034 1.19056E-06	6.0370 2.90016E-06	3 ALPHAS, REF B E, MEV. DK FRACTION
5.4263 4.00320E-04	5.9652 3.89977E-02	6.0460 1.00022E-06	6.2797 2.78071E-04
5.4887 8.30665E-03	6.0000 5.98229E-03	6.0840 2.50013E-06	6.3045 2.51269E-03
5.5292 5.80464E-03	6.0720 3.98819E-03	6.1000 3.40018E-05	6.3405 6.03046E-06
96-CM-246	6.1395 1.29616E-02	6.1220 7.80042E-06	6.3561 4.69030E-05
2 ALPHAS, REF A E, MEV. DK FRACTION	6.1940 2.39292E-02	6.1660 1.50408E-04	6.3791 1.94315E-04
5.3430 2.09945E-01	98-CF-250	6.2110 3.90021E-04	7.1889 8.50349E-01
5.3860 7.89793E-01	4 ALPHAS, REF A E, MEV. DK FRACTION	6.2170 1.50008E-05	99-ES-255
96-CM-247	5.7367 9.99130E-05	6.2300 1.20006E-06	3 ALPHAS, REF B E, MEV. DK FRACTION
7 ALPHAS, REF A E, MEV. DK FRACTION	5.8900 2.39739E-03	6.2500 1.50024E-04	6.2797 2.00000E-03
6.0308 8.34274E-01	5.9891 1.61859E-01	6.2660 8.00043E-06	6.2137 2.00000E-03
96-CM-248	6.4080 1.30007E-04	6.3250 4.00022E-06	6.2609 7.84000E-03
2 ALPHAS, REF A E, MEV. DK FRACTION	6.4320 6.10033E-04	6.3540 8.20044E-05	6.2996 7.01600E-02
5.8140 4.70000E-02	6.4800 8.50046E-04	6.4800 8.50046E-04	100-FM-255
5.8680 7.10000E-01	98-CF-251	6.4980 2.60014E-03	6 ALPHAS, REF B E, MEV. DK FRACTION
5.9410 1.60000E-02	14 ALPHAS, REF A E, MEV. DK FRACTION	6.5400 8.50046E-03	6.2797 2.00000E-03
5.9830 2.00000E-02	5.5010 3.04569E-03	6.5520 1.10038E-03	6.2137 2.00000E-03
5.1450 1.20000E-02	5.5560 1.52284E-02	6.5920 6.60036E-02	6.8069 1.10375E-03
5.2100 5.70000E-02	5.6030 2.03046E-03	6.5940 8.00043E-03	6.22156E-03
5.2650 1.38000E-01	5.6320 4.56853E-02	6.6240 8.00043E-03	6.9534 5.01706E-02
5.6480 3.55330E-02	6.6327 8.98048E-01	6.6327 8.98048E-01	6.9829 1.30444E-03
5.6770 5.52299E-01	98-CF-254	7.0225 9.37186E-01	7.0225 9.37186E-01
5.7380 1.01523E-02	2 ALPHAS, REF B E, MEV. DK FRACTION	7.0800 4.01365E-03	98-CF-254
5.7620 8.85781E-02	5.7973 5.270005E-04	100-FM-256	1 ALPHA, REF B E, MEV. DK FRACTION
5.7930 8.30466E-02	5.8366 2.57300E-03	5.7973 5.270005E-04	6.9152 1.00000E+00
5.8140 2.63966E-02	5.8366 2.57300E-03	5.8366 2.57300E-03	100-FM-257
5.8520 8.78173E-01	5.8520 8.78173E-01	5 ALPHAS, REF B E, MEV. DK FRACTION	6.3467 3.01811E-03
5.9430 6.09137E-03	5.9430 6.09137E-03	6.4410 2.01207E-02	6.4410 2.01207E-02
6.0140 1.21827E-01	6.0740 2.74112E-02	6.5199 9.35614E-01	6.5199 9.35614E-01
6.2241 1.80000E-01		6.8965 3.52113E-02	6.8965 3.52113E-02
6.2663 8.20000E-01		6.7572 6.03622E-03	6.7572 6.03622E-03

REFERENCE A: LNDFB/B-V
 REFERENCE B: TABLE OF ISOTOPES, SEVENTH EDITION

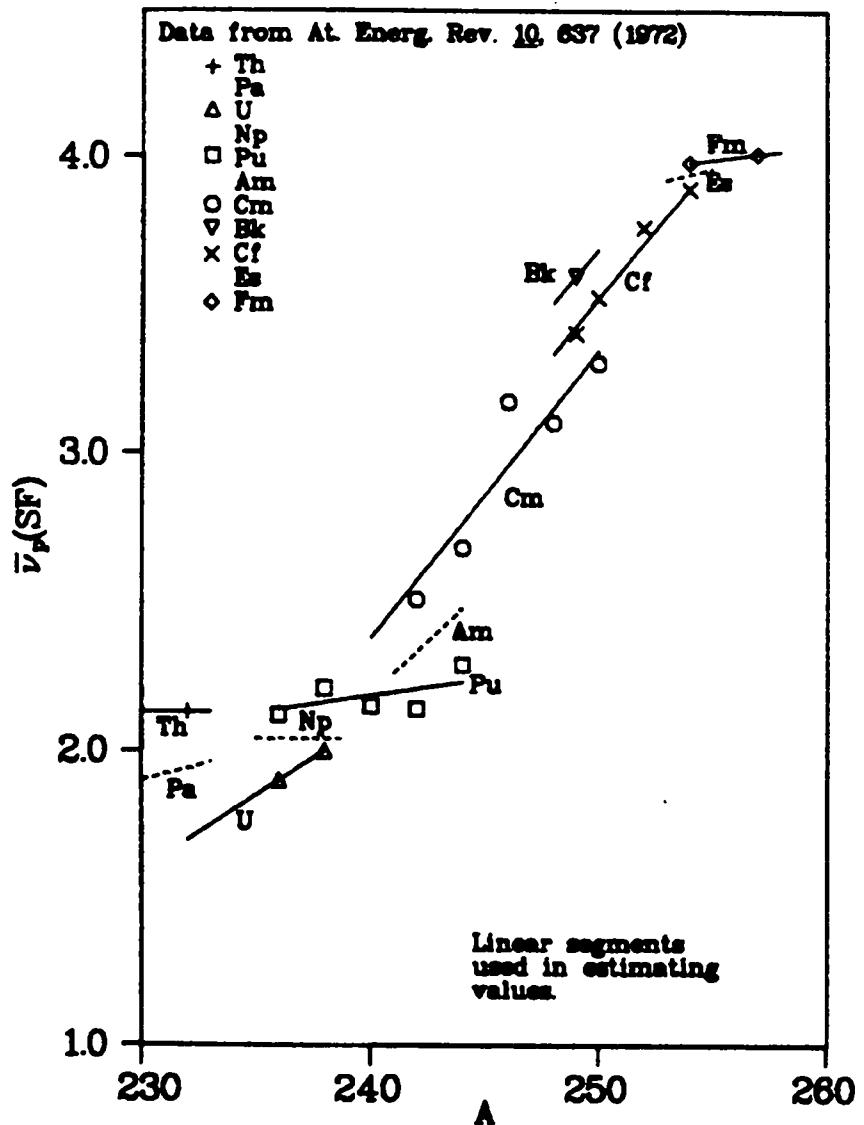


Fig. 4.
Values of $\bar{\nu}_p(\text{SF})$.

TABLE VIII
SPONTANEOUS-FISSION NEUTRON PRODUCTION BY ACTINIDE DECAY

NUCLIDE	PROMPT	NU-BAR VALUES		SPONTANEOUS FISSION BRANCHING	NEUTRONS PER NUCLIDE DECAY
		DELAYED	TOTAL		
90-TH-230	2.13	.01	2.14	5.330-13 A	1.14 -12
91-PA-231	1.92	.01	1.93	2.980-12 A	5.75 -12
90-TH-232	2.130+.200 B	.01	2.14	1.410-11 A	3.02 -11
92-U-232	1.70	.01	1.71	9.000-13 C	1.54 -12
92-U-233	1.75	.01	1.76	1.300-12 C	2.29 -12
92-U-234	1.80	.01	1.81	1.200-11 C	2.17 -11
92-U-235	1.85	.01	1.86	2.011-09 A	3.74 -09
92-U-236	1.900+.050 B	.01	1.91	1.200-09 C	2.29 -09
94-PU-236	2.120+.130 B	.01	2.13	8.100-10 C	1.73 -09
93-NP-237	2.04	.01	2.05	2.140-12 A	4.39 -12
92-U-238	2.000+.030 B	.01	2.01	5.450-07 C	1.095-06
94-PU-238	2.210+.130 B	.01	2.22	1.840-09 C	4.08 -09
94-PU-239	2.15	.01	2.16	4.400-12 C	9.37 -12
94-PU-240	2.151+.006 B	.01	2.16	5.000-08 C	1.08 -07
96-CM-240	2.38	.01	2.39	3.860-08 A	9.23 -08
95-RM-241	2.26	.01	2.27	4.100-12 C	9.31 -12
94-PU-242	2.141+.190 B	.01	2.15	5.500-06 C	1.18 -05
95-RM-242M	2.33	.01	2.34	1.600-10 C	3.74 -10
96-CM-242	2.510+.060 B	.01	2.52	6.800-08 C	1.71 -07
95-RM-243	2.41	.01	2.42	2.200-10 C	5.32 -10
94-PU-244	2.290+.190 B	.01	2.30	1.250-03 C	2.88 -03
96-CM-244	2.681+.011 B	.01	2.69	1.347-06 C	3.62 -06
96-CM-246	3.170+.220 B	.01	3.18	2.614-04 C	8.31 -04
96-CM-248	3.100+.090 B	.01	3.11	8.260-02 C	2.569-01
98-CF-248	3.33	.01	3.34	2.850-05 A	9.52 -05
97-BK-249	3.590+.160 B	.01	3.60	4.600-10 C	1.66 -09
98-CF-249	3.400+.400 B	.01	3.41	5.020-09 A	1.71 -08
96-CM-250	3.300+.080 B	.01	3.31	7.000-01 D	2.32 +00
98-CF-250	3.520+.090 B	.01	3.53	3.092-02 C	2.72 -03
98-CF-252	3.756+.012 B	.009 B	3.765+.010 B	3.092-02 C	1.164-01
99-ES-253	3.92	.01	3.93	8.700-08 C	3.42 -07
98-CF-254	3.890+.050 B	.01	3.890+.050 E	9.969-01 A	3.88 +00
99-ES-254	3.94	.01	3.95	3.020-08 A	1.19 -07
99-ES-254M	3.94	.01	3.95	4.500-08 A	1.78 -07
100-FM-254	3.980+.140 B	.01	3.96 +.14 F	5.900-04 A	2.34 -03
99-ES-255	3.96	.01	3.97	4.000-05 A	1.59 -04
100-FM-255	3.99	.01	3.73 +.18 F	2.290-07 A	8.54 -07
100-FM-256	4.00	.01	4.01	9.190-01 A	3.69 +00
100-FM-257	4.010+.130 B	.01	3.85 +.05 G	2.100-03 A	8.09 -03
100-FM-258	4.02	.01	4.03	1.000+00 A	4.03 +00

DATA REFERENCES USED
A=TABLE OF ISOTOPES, SEVENTH EDITION
B=MAHEPO AND KONSHIM, ATOMIC ENERGY REV. 10,637-756 (1972)
C=ENDF/B-V
D=A.TOBIAZ, U.K., PRIVATE COMMUNICATION
E=C.J.DORTH, NUCL.SCI.ENG.43,54 (1971)
F=Y.A.LAZAPEV, ATOMIC ENERGY REV.15,75 (1977)
G=D.C.HOFFMAN ET AL., PHYS.REV.C21,637 (1980)
ADDITIONAL REFERENCES SURVEYED
J.W.BOLDEMAN, IN NEUTRON STD. REF.DATA, I.A.E.A. VIENNA (1974)
J.P.BALAGNA ET AL., PHYS.REV.LETT.26,145 (1971)

PROMPT NU-BAR VALUES GIVEN WITHOUT REFERENCE HAVE BEEN ESTIMATED
FROM THE VALUES OF REFERENCE B. DELAYED NU-BAR VALUES GIVEN
WITHOUT REFERENCE HAVE BEEN ARBITRARILY ASSUMED.

IV. CALCULATION OF THE THICK-TARGET NEUTRON-PRODUCTION FUNCTION $P_i(E_\alpha)$

The neutron-production function $P_i(E_\alpha)$ defined by Eqs. (6) and (9) gives the contribution from reactions with nuclide i to the probability of neutron production by a decay alpha particle of energy E_α emitted within the material. The POFEAL code calculates values of P_i OF E -ALPHA using the algorithm

$$P(J) = 1.E + 6 * \frac{N_i}{N} \sum_{j=2}^J \frac{[\sigma_i(j-1) + \sigma_i(j)]/2}{[\epsilon(j-1) + \epsilon(j)]/2} [E(j) - E(j-1)] , \quad (25)$$

where

N_i is the atom density of nuclide i (atoms/cm³),

N is the total atom density (atoms/cm³),

E_j is the j th regular energy point at or above the cross-section threshold (MeV),

$\sigma_i(j)$ is the value of the (α, n) cross section of nuclide i at E_j (mb),

$\epsilon(j)$ is the value of the stopping cross section (eV/10¹⁵ atoms/cm²),

and the leading quantity of 1×10^6 is required because of the units of σ , ϵ , and E .

The 170 O and 180 O contributions to the (α, n) neutron-production rate are given in Tables IX-XII for each of the four fuel compositions given in Table I. Values for the four compositions at any energy differ by less than 4%. The 170 O and 180 O contributions to (α, n) neutron production in spent UO₂ fuel are shown in Fig. 5.

V. RESULTS

The half-lives, average decay energies, and spent UO₂ fuel neutron-production values $R_k(\alpha, n)$, $R_k(SF)$, and R_k for each of the actinide nuclides k are given in Table XIII. Values of $R_k(SF)$ are repeated from Table VIII. Values of $R_k(\alpha, n)$ were obtained using the alpha spectra data of Table VII and $P(E_\alpha)$ values given in Table XI for $^{17,18}O(\alpha, n)$ in spent UO₂ fuel.

TABLE IX

 $^{17,18}\text{O}(\alpha, n)$ NEUTRON PRODUCTION IN CLEAN ThO_2 FUEL BY ALPHA PARTICLES BELOW 10 MeV

E, MEV	NEUTRONS-PER-ALPHA*			NEUTRONS-PER-ALPHA*			NEUTRONS-PER-ALPHA*			NEUTRONS-PER-ALPHA*			NEUTRONS-PER-ALPHA*			NEUTRONS-PER-ALPHA*			
	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL
0.000	0.	0.	0.	2.824	4.7-11	4.7-10	5.22-10	4.624	9.0-10	1.08-8	1.175-8	6.424	3.01-9	3.43-8	3.733-8	8.224	6.61-9	7.05-8	7.713-8
1.130	0.	0.	0.	2.849	4.8-11	4.9-10	5.35-10	6.649	9.2-10	1.12-8	1.210-8	6.449	3.05-9	3.47-8	3.779-8	8.244	6.67-9	7.11-8	7.778-8
1.181	0.	0.	0.	2.874	5.0-11	5.2-10	5.68-10	6.674	9.4-10	1.15-8	1.235-8	6.474	3.09-9	3.52-8	3.826-8	8.274	6.71-9	7.17-8	7.844-8
1.161	0.	0.	0.	2.899	5.2-11	5.5-10	6.31-10	6.699	9.5-10	1.18-8	1.276-8	6.499	3.13-9	3.56-8	3.872-8	8.294	6.79-9	7.23-8	7.908-8
1.186	0.	0.	0.	2.924	5.4-11	5.6-10	7.21-10	7.724	9.7-10	1.20-8	1.300-8	6.524	3.17-9	3.60-8	3.919-8	8.324	6.86-9	7.29-8	7.974-8
1.216	0.	0.	0.	2.949	5.6-11	7.5-10	8.12-10	7.759	9.9-10	1.22-8	1.323-8	6.549	21-9	3.65-8	3.966-8	8.349	6.92-9	7.35-8	8.044-8
1.236	0.	0.	0.	2.974	7.0-11	7.7-10	8.42-10	7.774	1.01-9	2.4-8	1.343-8	6.574	25-9	3.69-8	4.014-8	8.374	6.98-9	7.41-8	8.108-8
1.266	0.	0.	0.	2.999	7.0-11	7.8-10	8.59-10	7.799	1.03-9	2.6-8	1.361-8	6.599	29-9	3.73-8	4.061-8	8.399	7.05-9	7.47-8	8.173-8
1.286	0.	0.	0.	3.024	8.3-11	7.9-10	8.78-10	8.824	8.05-9	1.28-8	1.380-8	6.624	33-9	3.78-8	4.109-8	8.424	7.11-9	7.53-8	8.240-8
1.309	0.	0.	0.	3.049	9.5-11	8.8-10	9.01-10	8.849	9.05-9	1.29-8	1.399-8	6.659	37-9	3.82-8	4.157-8	8.449	7.18-9	7.59-8	8.377-8
1.319	1.	0.	0.	3.074	9.7-11	10.	9.27-10	8.874	9.7-11	1.28-8	1.425-8	6.674	41-9	3.86-8	4.206-8	8.474	7.24-9	7.65-8	8.374-8
1.320	2.	0.	0.	3.099	9.8-11	10.	9.52-10	8.899	9.9-10	1.25-8	1.47-8	6.699	46-9	3.91-8	4.255-8	8.499	7.31-9	7.71-8	8.442-8
1.324	6.	0.	0.	1.124	1.0-11	10.	9.81-10	9.924	1.12-9	1.48-8	1.482-8	6.724	50-9	3.95-8	4.303-8	8.524	7.37-9	7.77-8	8.510-8
1.326	6.	0.	0.	1.149	1.0-11	10.	1.024-9	9.949	1.12-9	1.48-8	1.512-8	6.749	54-9	4.00-8	4.353-8	8.549	8.44-9	8.81-8	8.578-8
1.328	6.	0.	0.	1.174	1.0-11	10.	1.087-9	9.974	1.15-9	1.48-8	1.543-8	6.774	59-9	4.04-8	4.402-8	8.574	8.51-9	8.90-8	8.647-8
1.329	9.	0.	0.	1.199	1.0-11	10.	1.180-9	9.999	1.17-9	1.48-8	1.570-8	6.799	64-9	4.09-8	4.452-8	8.599	8.57-9	8.96-8	8.713-8
1.342	4.	0.	0.	2.240	1.0-11	10.	1.269-9	8.024	2.05-9	1.78-8	1.700-8	7.000	68-9	5.52-8	5.502-8	8.624	7.64-9	8.02-8	8.781-8
1.347	4.	0.	0.	2.265	1.0-11	10.	1.302-9	8.049	2.08-9	1.78-8	1.705-8	7.024	72-9	5.56-8	5.502-8	8.649	7.71-9	8.13-8	8.854-8
1.352	4.	0.	0.	2.290	1.0-11	10.	1.342-9	8.074	2.12-9	1.78-8	1.710-8	7.050	76-9	5.60-8	5.522-8	8.674	7.75-9	8.17-8	8.894-8
1.354	4.	0.	0.	2.315	1.0-11	10.	1.382-9	8.109	2.16-9	1.78-8	1.715-8	7.074	80-9	5.64-8	5.552-8	8.699	7.80-9	8.21-8	8.934-8
1.356	4.	0.	0.	2.340	1.0-11	10.	1.422-9	8.144	2.20-9	1.78-8	1.720-8	7.100	84-9	5.68-8	5.582-8	8.724	7.84-9	8.25-8	8.974-8
1.359	4.	0.	0.	2.365	1.0-11	10.	1.462-9	8.174	2.24-9	1.78-8	1.725-8	7.124	88-9	5.72-8	5.602-8	8.754	7.88-9	8.29-8	8.974-8
1.361	4.	0.	0.	2.390	1.0-11	10.	1.502-9	8.209	2.28-9	1.78-8	1.730-8	7.148	92-9	5.76-8	5.642-8	8.784	7.92-9	8.33-8	8.974-8
1.363	4.	0.	0.	2.415	1.0-11	10.	1.542-9	8.244	2.32-9	1.78-8	1.735-8	7.172	96-9	5.80-8	5.682-8	8.814	7.96-9	8.37-8	8.974-8
1.365	4.	0.	0.	2.440	1.0-11	10.	1.582-9	8.274	2.36-9	1.78-8	1.740-8	7.196	100-9	5.84-8	5.722-8	8.844	8.00-9	8.41-8	8.974-8
1.367	4.	0.	0.	2.465	1.0-11	10.	1.622-9	8.309	2.40-9	1.78-8	1.745-8	7.220	104-9	5.88-8	5.762-8	8.874	8.04-9	8.45-8	8.974-8
1.369	4.	0.	0.	2.490	1.0-11	10.	1.662-9	8.344	2.44-9	1.78-8	1.750-8	7.244	108-9	5.92-8	5.802-8	8.904	8.08-9	8.50-8	8.974-8
1.371	4.	0.	0.	2.515	1.0-11	10.	1.702-9	8.379	2.48-9	1.78-8	1.755-8	7.268	112-9	5.96-8	5.842-8	8.934	8.12-9	8.54-8	8.974-8
1.373	4.	0.	0.	2.540	1.0-11	10.	1.742-9	8.414	2.52-9	1.78-8	1.760-8	7.292	116-9	6.00-8	5.882-8	8.964	8.16-9	8.58-8	8.974-8
1.375	4.	0.	0.	2.565	1.0-11	10.	1.782-9	8.449	2.56-9	1.78-8	1.765-8	7.316	120-9	6.04-8	5.922-8	8.994	8.20-9	8.62-8	8.974-8
1.377	4.	0.	0.	2.590	1.0-11	10.	1.822-9	8.484	2.60-9	1.78-8	1.770-8	7.340	124-9	6.08-8	5.962-8	9.024	8.24-9	8.66-8	8.974-8
1.379	4.	0.	0.	2.615	1.0-11	10.	1.862-9	8.519	2.64-9	1.78-8	1.775-8	7.364	128-9	6.12-8	6.002-8	9.054	8.28-9	8.70-8	8.974-8
1.381	4.	0.	0.	2.640	1.0-11	10.	1.902-9	8.554	2.68-9	1.78-8	1.780-8	7.388	132-9	6.16-8	6.042-8	9.084	8.32-9	8.74-8	8.974-8
1.383	4.	0.	0.	2.665	1.0-11	10.	1.942-9	8.589	2.72-9	1.78-8	1.785-8	7.412	136-9	6.20-8	6.082-8	9.114	8.36-9	8.78-8	8.974-8
1.385	4.	0.	0.	2.690	1.0-11	10.	1.982-9	8.624	2.76-9	1.78-8	1.790-8	7.436	140-9	6.24-8	6.122-8	9.144	8.40-9	8.82-8	8.974-8
1.387	4.	0.	0.	2.715	1.0-11	10.	2.022-9	8.659	2.80-9	1.78-8	1.795-8	7.460	144-9	6.28-8	6.162-8	9.174	8.44-9	8.86-8	8.974-8
1.389	4.	0.	0.	2.740	1.0-11	10.	2.062-9	8.694	2.84-9	1.78-8	1.800-8	7.484	148-9	6.32-8	6.202-8	9.204	8.48-9	8.90-8	8.974-8
1.391	4.	0.	0.	2.765	1.0-11	10.	2.102-9	8.729	2.88-9	1.78-8	1.805-8	7.508	152-9	6.36-8	6.242-8	9.234	8.52-9	8.94-8	8.974-8
1.393	4.	0.	0.	2.790	1.0-11	10.	2.142-9	8.764	2.92-9	1.78-8	1.810-8	7.532	156-9	6.40-8	6.282-8	9.264	8.56-9	8.98-8	8.974-8
1.395	4.	0.	0.	2.815	1.0-11	10.	2.182-9	8.809	2.96-9	1.78-8	1.815-8	7.556	160-9	6.44-8	6.322-8	9.294	8.60-9	9.02-8	8.974-8
1.397	4.	0.	0.	2.840	1.0-11	10.	2.222-9	8.844	3.00-9	1.78-8	1.820-8	7.580	164-9	6.48-8	6.362-8	9.324	8.64-9	9.06-8	8.974-8
1.399	4.	0.	0.	2.865	1.0-11	10.	2.262-9	8.879	3.04-9	1.78-8	1.825-8	7.604	168-9	6.52-8	6.402-8	9.354	8.68-9	9.10-8	8.974-8
1.401	4.	0.	0.	2.890	1.0-11	10.	2.302-9	9.014	3.08-9	1.78-8	1.830-8	7.628	172-9	6.56-8	6.442-8	9.384	8.72-9	9.14-8	8.974-8
1.403	4.	0.	0.	2.915	1.0-11	10.	2.342-9	9.049	3.12-9	1.78-8	1.835-8	7.652	176-9	6.60-8	6.482-8	9.414	8.76-9	9.18-8	8.974-8
1.405	4.	0.	0.	2.940	1.0-11	10.	2.382-9	9.084	3.16-9	1.78-8	1.840-8	7.676	180-9	6.64-8	6.522-8	9.444	8.80-9	9.22-8	8.974-8
1.407	4.	0.	0.	2.965	1.0-11	10.	2.422-9	9.119	3.20-9	1.78-8	1.845-8	7.700	184-9	6.68-8	6.562-8	9.474	8.84-9	9.26-8	8.974-8
1.409	4.	0.	0.	2.990	1.0-11	10.	2.462-9	9.154	3.24-9	1.78-8	1.850-8	7.724	188-9	6.72-8	6.602-8	9.504	8.88-9	9.30-8	8.974-8
1.411	4.	0.	0.	3.015	1.0-11	10.	2.502-9	9.189	3.28-9	1.78-8	1.855-8	7.748	192-9	6.76-8	6.642-8	9.534	8.92-9	9.34-8	8.974-8
1.413	4.	0.	0.	3.040	1.0-11	10.	2.542-9	9.224	3.32-9	1.78-8	1.860-8	7.772	196-9	6.80-8	6.682-8	9.564	8.96-9	9.38-8	8.974-8
1.415	4.	0.	0.	3.065	1.0-11	10.	2.582-9	9.259	3.36-9	1.78-8	1.865-8	7.796	200-9	6.84-8	6.722-8	9.594	9.00-9	9.42-8	8.974-8
1.417	4.	0.	0.	3.090	1.0-11	10.	2.622-9	9.294	3.40-9	1.78-8	1.870-8	7.820	204-9	6.88-8	6.762-8	9.624	9.04-9	9.46-8	8.974-8

TABLE X
 $^{17,18}\text{O}(\alpha, n)$ NEUTRON PRODUCTION IN CLEAN UO_2 FUEL BY ALPHA PARTICLES BELOW 10 MeV

NEUTRONS-PER-ALPHA			**NEUTRONS-PER-ALPHA**			**NEUTRONS-PER-ALPHA**			**NEUTRONS-PER-ALPHA**			**NEUTRONS-PER-ALPHA**			**NEUTRONS-PER-ALPHA**				
E, MEV	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL	E, MEV	0-17	0-18	TOTAL
0.000	0.	0.	0.	2.822	4.7-11	4.7-10	5.17-10	4.624	8.9-10	1.07-8	1.158-8	6.424	9.8-9	3.27-8	3.667-8	6.224	6.48-2	6.91-8	7.557-8
1.139	0.	0.	0.	2.849	4.8-11	4.8-10	5.30-10	4.649	9.1-10	1.10-8	1.192-8	6.449	9.9-9	3.41-8	3.712-8	6.249	6.54-2	6.97-8	7.620-8
1.141	0.	0.	0.	2.874	4.9-11	5.1-10	5.63-10	4.674	9.2-10	1.13-8	1.226-8	6.474	3.03-9	3.45-8	3.757-8	6.274	6.60-2	7.02-8	7.684-8
1.161	0.0-0	1.5-14	1.53-14	2.899	5.2-11	5.7-10	6.25-10	4.699	9.4-10	1.16-8	1.257-8	6.499	3.07-9	3.50-8	3.803-8	6.299	6.66-2	7.06-8	7.748-8
1.186	0.0-0	1.6-14	1.78-14	2.929	5.7-11	6.6-10	7.14-10	4.724	9.6-10	1.18-8	1.281-8	6.524	3.11-9	3.54-8	3.849-8	6.324	6.72-2	7.14-8	7.812-8
1.211	0.0-0	2.6-14	2.63-14	2.949	6.5-11	7.4-10	8.04-10	4.749	9.8-10	1.21-8	1.303-8	6.549	3.15-9	3.58-8	3.895-8	6.349	6.78-2	7.20-8	7.876-8
1.226	0.0-0	2.6-14	2.63-14	2.974	7.0-11	7.8-10	8.33-10	4.774	1.0-10	1.22-8	1.321-8	6.574	3.19-9	3.62-8	3.942-8	6.374	6.84-2	7.26-8	7.941-8
1.261	0.0-0	3.0-14	3.04-14	2.999	7.4-11	7.8-10	8.50-10	4.799	1.01-9	1.24-8	1.341-8	6.599	3.23-9	3.60-8	3.988-8	6.399	6.90-2	7.32-8	8.006-8
1.266	0.0-0	3.1-14	3.21-14	3.024	8.2-11	7.9-10	8.69-10	4.824	1.02-9	1.26-8	1.359-8	6.624	3.27-9	3.71-8	4.035-8	6.424	6.97-2	7.37-8	8.071-8
1.269	0.0-0	3.2-14	3.21-14	3.049	9.4-11	8.0-10	8.94-10	4.849	1.04-9	1.27-8	1.371-8	6.649	3.31-9	3.75-8	4.086-8	6.449	7.03-2	7.45-8	8.137-8
1.119	1.0-16	5.7-14	5.87-14	3.074	1.1-10	8.1-10	9.17-10	4.874	1.06-9	1.30-8	1.403-8	6.674	3.35-9	3.79-8	4.130-8	6.474	7.09-2	7.44-8	8.203-8
1.119	1.0-15	5.7-14	5.87-14	3.094	1.2-10	8.2-10	9.42-10	4.899	1.07-9	1.33-8	1.434-8	6.699	3.40-9	3.84-8	4.177-8	6.499	7.16-2	7.55-8	8.269-8
1.224	1.0-15	5.8-14	5.87-14	3.124	1.3-10	8.4-10	9.71-10	4.924	1.09-9	1.35-8	1.459-8	6.724	3.44-9	3.88-8	4.225-8	6.524	7.22-2	7.61-8	8.335-8
1.244	6.0-16	8.8-14	1.02-14	3.149	1.4-10	8.8-10	1.013-8	4.949	1.10-9	1.36-8	1.489-8	6.749	3.48-9	3.93-8	4.273-8	6.589	7.29-2	7.67-8	8.401-8
1.244	6.0-15	9.1-14	1.07-14	3.174	1.4-10	9.3-10	1.076-8	4.974	1.12-9	1.41-8	1.519-8	6.774	3.52-9	3.97-8	4.322-8	6.574	7.35-2	7.73-8	8.468-8
1.269	1.0-15	1.0-14	1.13-14	3.199	1.5-10	9.6-10	1.167-8	5.009	1.16-9	1.43-8	1.546-8	6.799	3.56-9	4.01-8	4.370-8	6.599	7.42-2	7.74-8	8.535-8
1.242	1.0-14	1.1-14	1.13-14	3.224	1.6-10	10-9	1.256-8	5.024	1.19-9	1.44-8	1.562-8	6.824	3.61-9	4.06-8	4.419-8	6.524	7.48-2	7.85-8	8.603-8
1.249	4.0-16	1.7-14	1.21-14	3.249	1.6-10	10-9	1.275-8	5.049	1.21-9	1.45-8	1.575-8	6.849	3.65-9	4.10-8	4.466-8	6.549	7.55-2	7.91-8	8.670-8
1.274	4.0-15	1.7-14	1.21-14	3.274	1.6-10	10-9	1.294-8	5.074	1.23-9	1.47-8	1.590-8	6.874	3.69-9	4.15-8	4.517-8	6.574	7.61-2	7.98-8	8.687-8
1.242	1.0-14	1.0-14	1.21-14	3.294	1.7-10	10-9	1.314-8	5.099	1.24-9	1.50-8	1.620-8	6.899	3.73-9	4.20-8	4.667-8	6.599	7.58-2	8.04-8	8.807-8
1.274	1.0-14	1.0-14	1.21-14	3.324	1.7-10	10-9	1.334-8	5.124	1.26-9	1.53-8	1.654-8	6.924	3.78-9	4.24-8	4.724	6.617-8	7.59-2	8.10-8	8.875-8
1.274	1.0-14	1.0-14	1.21-14	3.349	1.7-10	10-9	1.354-8	5.149	1.28-9	1.56-8	1.685-8	6.949	3.82-9	4.28-8	4.749	6.652-8	8.02-2	8.16-8	8.944-8
1.274	1.0-14	1.0-14	1.21-14	3.374	1.7-10	10-9	1.374-8	5.174	1.30-9	1.59-8	1.719-8	6.974	3.86-9	4.32-8	4.774	6.686-8	8.22-2	8.22-8	9.013-8
1.274	1.0-14	1.0-14	1.21-14	3.399	1.7-10	10-9	1.394-8	5.204	1.32-9	1.62-8	1.752-8	7.004	3.90-9	4.36-8	4.804	6.724-8	8.27-2	8.27-8	9.043-8
1.274	1.0-14	1.0-14	1.21-14	3.424	1.7-10	10-9	1.414-8	5.229	1.34-9	1.65-8	1.785-8	7.024	3.94-9	4.42-8	4.819	6.744-8	8.31-2	8.31-8	9.053-8
1.274	1.0-14	1.0-14	1.21-14	3.449	1.7-10	10-9	1.434-8	5.249	1.36-9	1.68-8	1.819-8	7.049	3.98-9	4.46-8	4.839	6.764-8	8.35-2	8.35-8	9.063-8
1.274	1.0-14	1.0-14	1.21-14	3.474	1.7-10	10-9	1.454-8	5.274	1.38-9	1.71-8	1.883-8	7.074	4.02-9	4.50-8	4.874	6.784-8	8.38-2	8.38-8	9.073-8
1.274	1.0-14	1.0-14	1.21-14	3.509	1.7-10	10-9	1.474-8	5.304	1.40-9	1.74-8	1.921-8	7.109	4.06-9	4.54-8	4.894	6.804-8	8.41-2	8.41-8	9.083-8
1.274	1.0-14	1.0-14	1.21-14	3.534	1.7-10	10-9	1.494-8	5.324	1.42-9	1.77-8	1.969-8	7.134	4.10-9	4.58-8	4.924	6.824-8	8.44-2	8.44-8	9.093-8
1.274	1.0-14	1.0-14	1.21-14	3.559	1.7-10	10-9	1.514-8	5.349	1.44-9	1.80-8	2.004-8	7.159	4.14-9	4.62-8	4.944	6.844-8	8.47-2	8.47-8	9.103-8
1.274	1.0-14	1.0-14	1.21-14	3.584	1.7-10	10-9	1.534-8	5.374	1.46-9	1.83-8	2.040-8	7.184	4.18-9	4.66-8	4.964	6.864-8	8.50-2	8.50-8	9.113-8
1.274	1.0-14	1.0-14	1.21-14	3.609	1.7-10	10-9	1.554-8	5.404	1.48-9	1.86-8	2.076-8	7.214	4.22-9	4.70-8	4.984	6.884-8	8.53-2	8.53-8	9.123-8
1.274	1.0-14	1.0-14	1.21-14	3.634	1.7-10	10-9	1.574-8	5.424	1.50-9	1.89-8	2.112-8	7.244	4.26-9	4.74-8	5.004	7.004-8	8.56-2	8.56-8	9.133-8
1.274	1.0-14	1.0-14	1.21-14	3.659	1.7-10	10-9	1.594-8	5.449	1.52-9	1.92-8	2.148-8	7.274	4.30-9	4.78-8	5.024	7.024-8	8.59-2	8.59-8	9.143-8
1.274	1.0-14	1.0-14	1.21-14	3.684	1.7-10	10-9	1.614-8	5.474	1.54-9	1.95-8	2.184-8	7.304	4.34-9	4.82-8	5.044	7.044-8	8.62-2	8.62-8	9.153-8
1.274	1.0-14	1.0-14	1.21-14	3.709	1.7-10	10-9	1.634-8	5.504	1.56-9	1.98-8	2.220-8	7.334	4.38-9	4.86-8	5.064	7.064-8	8.65-2	8.65-8	9.163-8
1.274	1.0-14	1.0-14	1.21-14	3.734	1.7-10	10-9	1.654-8	5.524	1.58-9	2.01-8	2.256-8	7.364	4.42-9	4.90-8	5.084	7.084-8	8.68-2	8.68-8	9.173-8
1.274	1.0-14	1.0-14	1.21-14	3.759	1.7-10	10-9	1.674-8	5.549	1.60-9	2.04-8	2.292-8	7.394	4.46-9	4.94-8	5.104	7.104-8	8.71-2	8.71-8	9.183-8
1.274	1.0-14	1.0-14	1.21-14	3.784	1.7-10	10-9	1.694-8	5.574	1.62-9	2.07-8	2.328-8	7.424	4.50-9	4.98-8	5.124	7.124-8	8.74-2	8.74-8	9.193-8
1.274	1.0-14	1.0-14	1.21-14	3.809	1.7-10	10-9	1.714-8	5.604	1.64-9	2.10-8	2.364-8	7.454	4.54-9	5.02-8	5.144	7.144-8	8.77-2	8.77-8	9.203-8
1.274	1.0-14	1.0-14	1.21-14	3.834	1.7-10	10-9	1.734-8	5.624	1.66-9	2.13-8	2.400-8	7.484	4.58-9	5.06-8	5.164	7.164-8	8.80-2	8.80-8	9.213-8
1.274	1.0-14	1.0-14	1.21-14	3.859	1.7-10	10-9	1.754-8	5.649	1.68-9	2.16-8	2.436-8	7.514	4.62-9	5.10-8	5.184	7.184-8	8.83-2	8.83-8	9.223-8
1.274	1.0-14	1.0-14	1.21-14	3.884	1.7-10	10-9	1.774-8	5.674	1.70-9	2.19-8	2.472-8	7.544	4.66-9	5.14-8	5.204	7.204-8	8.86-2	8.86-8	9.233-8
1.274	1.0-14	1.0-14	1.21-14	3.909	1.7-10	10-9	1.794-8	5.704	1.72-9	2.22-8	2.508-8	7.574	4.70-9	5.18-8	5.224	7.224-8	8.89-2	8.89-8	9.243-8
1.274	1.0-14	1.0-14	1.21-14	3.934	1.7-10	10-9	1.814-8	5.734	1.74-9	2.25-8	2.544-8	7.604	4.74-9	5.22-8	5.244	7.244-8	8.92-2	8.92-8	9.253-8
1.274	1.0-14	1.0-14	1.21-14	3.959	1.7-10	10-9	1.834-8	5.764	1.76-9	2.28-8	2.584-8	7.634	4.78-9	5.26-8	5.264	7.264-8	8.95-2	8.95-8	9.263-8
1.274	1.0-14	1.0-14	1.21-14	3.984	1.7-10	10-9	1.854-8	5.794	1.78-9	2.31-8	2.624-8	7.664	4.82-9	5.30-8	5.284	7.284-8	8.98-2	8.98-8	9.273-8
1.274	1.0-14	1.0-14																	

TABLE XI

 $^{17,18}\text{O}(\alpha, n)$ NEUTRON PRODUCTION IN SPENT UO_2 FUEL BY ALPHA PARTICLES BELOW 10 MeV

NEUTRONS-PER-ALPHA			NEUTRONS-PER-ALPHA			NEUTRONS-PER-ALPHA			NEUTRONS-PER-ALPHA			NEUTRONS-PER-ALPHA			
E, MeV	U-17	U-18	TOTAL	E, MeV	0-17	0-18	TOTAL	E, MeV	0-17	0-18	TOTAL	E, MeV	0-17	0-18	TOTAL
0.000	0.	0.	0.	2.824	4.6-11	4.6-10	5.11-10	4.624	8.8-10	1.06-8	1.14-8	6.424	22.9	33.8	5.62-8
1.139	0.	0.	0.	2.849	4.7-11	4.8-10	5.24-10	4.649	9.0-11	1.09-8	1.17-8	6.449	22.9	33.8	5.64-8
1.161	0.0-0	1.5-14	1.52-14	2.869	5.2-11	5.7-10	5.55-10	4.674	8.6-11	1.17-8	1.24-8	6.474	22.9	33.8	5.67-8
1.186	0.0-0	1.8-14	1.76-14	2.894	5.6-11	6.1-10	5.55-10	4.699	8.3-11	1.15-8	1.24-8	6.499	22.9	33.8	5.69-8
1.211	0.0-0	2.6-14	2.59-14	2.949	6.4-11	7.3-10	7.95-10	4.749	9.7-10	1.19-8	1.28-8	6.749	22.9	33.8	5.80-8
1.236	0.0-0	2.6-14	2.59-14	2.974	6.9-11	7.5-10	8.23-10	4.774	9.8-11	1.21-8	1.30-8	6.774	22.9	33.8	5.80-8
1.261	0.0-0	2.6-14	2.59-14	2.999	7.3-11	7.7-10	8.40-10	4.799	1.0-10	1.23-8	1.32-8	6.799	22.9	33.8	5.80-8
1.286	0.0-0	3.0-14	3.01-14	3.024	7.1-11	7.8-10	8.59-10	4.824	1.0-10	1.24-8	1.33-8	6.824	22.9	33.8	5.81-8
1.309	0.0-0	3.2-14	3.17-14	3.049	9.3-11	9.7-10	8.81-10	4.849	0.3-9	1.26-8	1.36-8	6.849	22.9	33.8	5.81-8
1.311	1.0-16	3.2-14	3.27-14	3.074	1.1-10	8.0-10	9.07-10	4.874	1.0-9	1.28-8	1.37-8	6.874	22.9	33.8	5.81-8
1.319	1.5-15	3.2-14	3.80-14	3.099	2.1-10	8.0-10	9.31-10	4.899	1.0-6	1.31-8	1.41-8	6.899	22.9	33.8	5.81-8
1.324	2.3-15	3.0-14	3.20-14	3.124	3.3-10	8.0-10	9.59-10	4.924	1.0-7	1.34-8	1.44-8	6.924	22.9	33.8	5.81-8
1.349	4.2-15	9.7-13	1.01-13	3.149	3.10-10	7-10	1.00-9	4.949	1.0-9	1.36-8	1.47-8	6.949	22.9	33.8	5.81-8
1.375	6.1-15	1.0-13	1.06-13	3.174	4.10-9	9.2-10	1.06-9	4.974	1.1-9	1.39-8	1.50-8	6.974	22.9	33.8	5.81-8
1.399	1.0-14	1.0-13	1.12-13	3.199	5.10-10	1.00-9	1.153-9	4.999	1.1-9	1.41-8	1.52-8	6.999	22.9	33.8	5.81-8
1.424	1.2-14	1.1-13	1.18-13	3.224	5.10-10	1.09-9	1.241-9	5.024	1.1-9	1.43-8	1.544-8	6.824	22.9	33.8	5.81-8
1.449	4.7-14	1.6-13	2.11-13	3.249	6.10-10	1.15-9	1.310-9	5.049	1.1-9	1.44-8	1.557-8	6.849	22.9	33.8	5.81-8
1.475	9.0-14	2.1-13	2.99-13	3.274	6.10-10	1.20-9	1.364-9	5.074	2.1-9	1.45-8	1.572-8	6.874	22.9	33.8	5.81-8
1.499	1.0-14	2.5-13	2.57-13	3.299	6.10-10	1.26-9	1.422-9	5.099	2.3-9	1.48-8	1.601-8	6.899	22.9	33.8	5.81-8
1.524	1.1-15	2.5-13	2.6-13	3.324	7.1-10	1.34-9	1.503-9	5.124	2.5-9	1.51-8	1.634-8	6.924	22.9	33.8	5.81-8
1.549	2.1-15	1.0-13	1.03-12	3.349	7.1-10	1.34-9	1.604-9	5.149	2.6-9	1.54-8	1.660-8	6.949	22.9	33.8	5.81-8
1.575	2.1-15	1.0-13	1.16-12	3.374	8.1-10	1.51-9	1.684-9	5.174	3.2-9	1.57-8	1.699-8	6.974	22.9	33.8	5.81-8
1.600	1.4-15	1.1-12	1.23-12	3.399	8.1-10	1.56-9	1.739-9	5.199	3.5-9	1.60-8	1.732-8	6.999	22.9	33.8	5.81-8
1.624	1.6-15	1.1-12	1.30-12	3.424	9.1-10	1.59-9	1.778-9	5.224	3.7-9	1.63-8	1.765-8	7.024	22.9	33.8	5.81-8
1.649	1.9-15	1.2-12	1.36-12	3.449	2.0-10	1.62-9	1.821-9	5.249	4.0-9	1.66-8	1.798-8	7.049	22.9	33.8	5.81-8
1.674	2.2-15	1.2-12	1.43-12	3.474	2.0-10	1.66-9	1.864-9	5.274	4.3-9	1.69-8	1.832-8	7.074	22.9	33.8	5.81-8
1.699	2.2-15	1.2-12	1.50-12	3.499	2.1-10	1.72-9	1.931-9	5.299	4.6-9	1.72-8	1.865-8	7.099	22.9	33.8	5.81-8
1.724	3.2-15	1.3-12	1.59-12	3.524	2.2-10	1.82-9	2.042-9	5.324	4.9-9	1.75-8	1.899-8	7.124	22.9	33.8	5.81-8
1.749	3.4-15	1.3-12	1.71-12	3.549	3.3-10	1.97-9	2.201-9	5.349	5.1-9	1.78-8	1.933-9	7.149	22.9	33.8	5.81-8
1.774	5.0-15	1.4-12	1.91-12	3.574	4.4-10	2.13-9	2.370-9	5.374	5.4-9	1.81-8	1.967-9	7.174	22.9	33.8	5.81-8
1.799	5.6-15	1.4-12	1.97-12	3.599	5.2-10	2.13-9	2.526-9	5.399	5.7-9	1.85-8	2.002-8	7.199	22.9	33.8	5.81-8
1.824	7.6-15	1.4-12	2.02-12	3.624	6.2-10	2.51-9	2.775-9	5.424	6.0-9	1.88-8	2.037-8	7.224	22.9	33.8	5.81-8
1.849	9.1-15	1.2-12	2.02-12	3.649	8.6-10	2.66-9	2.939-9	5.449	6.3-9	1.91-8	2.072-8	7.249	22.9	33.8	5.81-8
1.875	1.1-15	1.2-12	2.02-12	3.674	3.0-10	2.79-9	3.086-9	5.474	6.5-9	1.94-8	2.107-8	7.274	22.9	33.8	5.81-8
1.899	1.2-15	1.2-12	2.02-12	3.699	3.1-10	2.92-9	3.227-9	5.499	6.8-9	1.97-8	2.143-8	7.299	22.9	33.8	5.81-8
1.924	1.6-15	1.2-12	2.02-12	3.724	3.1-10	3.04-9	3.362-9	5.524	7.1-9	2.01-8	2.175-8	7.324	22.9	33.8	5.81-8
1.949	1.7-15	1.2-12	2.02-12	3.749	3.1-10	3.16-9	3.503-9	5.549	7.4-9	2.04-8	2.214-8	7.349	22.9	33.8	5.81-8
1.974	2.0-15	1.2-12	2.02-12	3.774	3.1-10	3.29-9	3.659-9	5.574	7.7-9	2.07-8	2.250-8	7.374	22.9	33.8	5.81-8
1.999	2.0-15	1.2-12	2.02-12	3.799	3.1-10	3.46-9	3.849-9	5.599	8.0-9	2.09-8	2.287-8	7.399	22.9	33.8	5.81-8
2.024	2.0-15	1.2-12	2.02-12	3.824	4.0-10	3.58-9	3.979-9	5.624	8.3-9	2.12-8	2.324-8	7.424	22.9	33.8	5.81-8
2.049	4.3-15	1.2-12	2.02-12	3.849	4.2-10	3.95-9	3.966-9	5.649	8.6-9	2.17-8	2.360-8	7.449	22.9	33.8	5.81-8
2.074	4.3-15	1.2-12	2.02-12	3.874	4.3-10	4.18-9	4.013-9	5.674	8.9-9	2.21-8	2.398-8	7.474	22.9	33.8	5.81-8
2.099	4.5-15	1.2-12	2.02-12	3.899	4.4-10	4.10-9	4.043-9	5.699	9.2-9	2.24-8	2.435-8	7.499	22.9	33.8	5.81-8
2.124	5.0-15	1.2-12	2.02-12	3.924	4.6-10	4.53-9	5.088-9	5.724	9.5-9	2.28-8	2.473-8	7.524	22.9	33.8	5.81-8
2.149	7.6-15	1.2-12	2.02-12	3.949	7.7-10	4.82-9	5.290-9	5.749	9.8-9	2.31-8	2.511-8	7.549	22.9	33.8	5.81-8
2.175	1.0-15	1.1-12	2.02-12	3.974	8.8-10	4.94-9	5.424-9	5.774	2.02-9	2.35-8	2.549-8	7.574	22.9	33.8	5.81-8
2.199	1.2-15	1.1-12	2.02-12	3.999	9.0-10	5.02-9	5.519-9	5.799	2.05-9	2.38-8	2.587-8	7.599	22.9	33.8	5.81-8
2.224	1.5-15	1.1-12	2.02-12	4.024	2.2-10	5.08-9	5.607-9	5.824	2.08-9	2.42-8	2.626-8	7.624	22.9	33.8	5.81-8
2.249	1.6-15	1.1-12	2.02-12	4.049	5.5-10	5.15-9	5.703-9	5.849	1.1-9	2.45-8	2.665-8	7.649	22.9	33.8	5.81-8
2.274	1.7-15	1.1-12	2.02-12	4.074	5.7-10	5.25-9	5.821-9	5.874	2.14-9	2.49-8	2.704-8	7.674	22.9	33.8	5.81-8
2.300	1.7-15	1.1-12	2.02-12	4.099	5.9-10	5.42-9	5.915-9	5.899	2.18-9	2.53-8	2.743-8	7.699	22.9	33.8	5.81-8
2.325	1.7-15	1.1-12	2.02-12	4.124	6.1-10	5.63-9	6.241-9	5.924	2.21-9	2.56-8	2.783-8	7.724	22.9	33.8	5.81-8
2.350	1.8-15	1.1-12	2.02-12	4.149	6.2-10	5.77-9	6.398-9	5.949	2.24-9	2.60-8	2.822-8	7.749	22.9	33.8	5.81-8
2.375	1.8-15	1.1-12	2.02-12	4.174	6.3-10	5.91-9	6.555-9	5.974	2.28-9	2.63-8	2.862-8	7.774	22.9	33.8	5.81-8
2.400	1.8-15	1.1-12	2.02-12	4.199	6.5-10	6.05-9	6.611-9	5.999	2.31-9	2.67-8	2.903-8	7.799	22.9	33.8	5.81-8
2.425	1.8-15	1.1-12	2.02-12	4.224	6.8-10	6.23-9	6.911-9	6.024	2.34-9	2.71-8	2.943-8	7.824	22.9	33.8	5.81-8
2.449	1.8-15	1.1-12	2.02-12	4.249	6.9-10	6.49-9	7.180-9	6.049	2.38-9	2.75-8	2.984-8	7.849	22.9	33.8	5.81-8
2.474	1.7-15	1.1-12	2.02-12	4.275	7.0-10	6.79-9	7.493-9	6.074	2.41-9	2.78-8	3.022-8	7.874	22.9	33.8	5.81-8
2.499	1.7-15	1.1-12	2.02-12	4.299	7.2-10	7.07-9	7.784-9	6.099	2.45-9	2.82-8	3.066-8	7.899	22.9	33.8	5.81-8
2.524	1.7-15	1.1-12	2.02-12	4.324	7.3-10	7.28-9	8.016-9	6.124	2.48-9	2.86-8	3.108-8	7.924	22.9	33.8	5.81-8
2.549	1.7-15	1.1-12	2.02-12	4.349	7.5-10	7.53-9	8.277-9	6.149	2.52-9	2.90-8	3.150-8	7.949	22.9	33.8	5.81-8
2.574															

TABLE XII

 $^{17,18}\text{O}(\alpha, n)$ NEUTRON PRODUCTION IN CLEAN (U, Pu)O₂ FUEL BY ALPHA PARTICLES BELOW 10 MeV

NEUTRONS-Per-ALPHA			NEUTRONS-Per-ALPHA			NEUTRONS-Per-ALPHA			NEUTRONS-Per-ALPHA			NEUTRONS-Per-ALPHA			
E, MeV	U-17	U-18	TOTAL	E, MeV	U-17	U-18	TOTAL	E, MeV	U-17	U-18	TOTAL	E, MeV	U-17	U-18	TOTAL
0.000	0.0	0.0	0.0	2.829	4.7-11	7-10	5-19-10	4.624	8-9-10	1-07-8	162-8	6.424	2-9-8	3-28-8	3.680-8
1.141	0.0	0.0	0.0	2.840	4.8-11	7-10	5-19-10	4.640	9-10-10	1-07-8	197-8	6.440	2-9-8	4-42-8	3.680-8
1.186	0.0	0.0	0.0	2.840	4.8-11	7-10	5-19-10	4.674	9-10-10	1-07-8	221-8	6.474	0-08-8	4-47-8	3.680-8
1.230	0.0	0.0	0.0	2.840	4.8-11	7-10	5-19-10	4.724	9-10-10	1-07-8	262-8	6.474	0-08-8	4-51-8	3.680-8
1.286	0.0	0.0	0.0	2.840	4.8-11	7-10	5-19-10	4.774	9-10-10	1-07-8	308-8	6.474	0-08-8	4-56-8	3.680-8
1.342	0.0	0.0	0.0	2.840	4.8-11	7-10	5-19-10	4.824	9-10-10	1-07-8	347-8	6.474	0-08-8	4-61-8	3.680-8
1.398	0.0	0.0	0.0	2.840	4.8-11	7-10	5-19-10	4.874	9-10-10	1-07-8	386-8	6.474	0-08-8	4-66-8	3.680-8
1.454	0.0	0.0	0.0	2.840	4.8-11	7-10	5-19-10	4.924	9-10-10	1-07-8	425-8	6.474	0-08-8	4-71-8	3.680-8
1.510	1.0	0.0	1.0	2.840	4.8-11	7-10	5-19-10	4.974	9-10-10	1-07-8	464-8	6.474	0-08-8	4-76-8	3.680-8
1.566	2.4	1.4	3.8	2.840	4.8-11	7-10	5-19-10	5.024	9-10-10	1-07-8	503-8	6.474	0-08-8	4-81-8	3.680-8
1.622	6.0	1.0	7.0	2.840	4.8-11	7-10	5-19-10	5.074	9-10-10	1-07-8	542-8	6.474	0-08-8	4-86-8	3.680-8
1.678	8.0	1.4	9.4	2.840	4.8-11	7-10	5-19-10	5.124	9-10-10	1-07-8	581-8	6.474	0-08-8	4-91-8	3.680-8
1.734	9.0	1.4	10.4	2.840	4.8-11	7-10	5-19-10	5.174	9-10-10	1-07-8	620-8	6.474	0-08-8	4-96-8	3.680-8
1.790	9.4	1.4	10.8	2.840	4.8-11	7-10	5-19-10	5.224	9-10-10	1-07-8	659-8	6.474	0-08-8	4-101-8	3.680-8
1.846	1.2	1.4	2.4	2.840	4.8-11	7-10	5-19-10	5.274	9-10-10	1-07-8	698-8	6.474	0-08-8	4-106-8	3.680-8
1.892	2.3	1.4	3.7	2.840	4.8-11	7-10	5-19-10	5.324	9-10-10	1-07-8	737-8	6.474	0-08-8	4-111-8	3.680-8
1.948	3.2	1.4	4.6	2.840	4.8-11	7-10	5-19-10	5.374	9-10-10	1-07-8	776-8	6.474	0-08-8	4-116-8	3.680-8
2.004	4.2	1.4	5.6	2.840	4.8-11	7-10	5-19-10	5.424	9-10-10	1-07-8	815-8	6.474	0-08-8	4-121-8	3.680-8
2.060	5.2	1.4	6.6	2.840	4.8-11	7-10	5-19-10	5.474	9-10-10	1-07-8	854-8	6.474	0-08-8	4-126-8	3.680-8
2.116	6.2	1.4	7.6	2.840	4.8-11	7-10	5-19-10	5.524	9-10-10	1-07-8	893-8	6.474	0-08-8	4-131-8	3.680-8
2.172	7.2	1.4	8.6	2.840	4.8-11	7-10	5-19-10	5.574	9-10-10	1-07-8	932-8	6.474	0-08-8	4-136-8	3.680-8
2.228	8.2	1.4	9.6	2.840	4.8-11	7-10	5-19-10	5.624	9-10-10	1-07-8	971-8	6.474	0-08-8	4-141-8	3.680-8
2.284	9.2	1.4	10.6	2.840	4.8-11	7-10	5-19-10	5.674	9-10-10	1-07-8	1010-8	6.474	0-08-8	4-146-8	3.680-8
2.340	1.2	1.4	2.4	2.840	4.8-11	7-10	5-19-10	5.724	9-10-10	1-07-8	1049-8	6.474	0-08-8	4-151-8	3.680-8
2.396	2.1	1.4	3.5	2.840	4.8-11	7-10	5-19-10	5.774	9-10-10	1-07-8	1088-8	6.474	0-08-8	4-156-8	3.680-8
2.452	3.1	1.4	4.5	2.840	4.8-11	7-10	5-19-10	5.824	9-10-10	1-07-8	1127-8	6.474	0-08-8	4-161-8	3.680-8
2.508	4.1	1.4	5.5	2.840	4.8-11	7-10	5-19-10	5.874	9-10-10	1-07-8	1166-8	6.474	0-08-8	4-166-8	3.680-8
2.564	5.1	1.4	6.5	2.840	4.8-11	7-10	5-19-10	5.924	9-10-10	1-07-8	1205-8	6.474	0-08-8	4-171-8	3.680-8
2.620	6.1	1.4	7.5	2.840	4.8-11	7-10	5-19-10	5.974	9-10-10	1-07-8	1244-8	6.474	0-08-8	4-176-8	3.680-8
2.676	7.1	1.4	8.5	2.840	4.8-11	7-10	5-19-10	6.024	9-10-10	1-07-8	1283-8	6.474	0-08-8	4-181-8	3.680-8
2.732	8.1	1.4	9.5	2.840	4.8-11	7-10	5-19-10	6.074	9-10-10	1-07-8	1322-8	6.474	0-08-8	4-186-8	3.680-8
2.788	9.1	1.4	10.5	2.840	4.8-11	7-10	5-19-10	6.124	9-10-10	1-07-8	1361-8	6.474	0-08-8	4-191-8	3.680-8
2.844	1.2	1.4	2.2	2.840	4.8-11	7-10	5-19-10	6.174	9-10-10	1-07-8	1400-8	6.474	0-08-8	4-196-8	3.680-8
2.899	2.1	1.4	3.5	2.840	4.8-11	7-10	5-19-10	6.224	9-10-10	1-07-8	1439-8	6.474	0-08-8	4-201-8	3.680-8
2.955	3.1	1.4	4.5	2.840	4.8-11	7-10	5-19-10	6.274	9-10-10	1-07-8	1478-8	6.474	0-08-8	4-206-8	3.680-8
3.011	4.1	1.4	5.5	2.840	4.8-11	7-10	5-19-10	6.324	9-10-10	1-07-8	1517-8	6.474	0-08-8	4-211-8	3.680-8
3.067	5.1	1.4	6.5	2.840	4.8-11	7-10	5-19-10	6.374	9-10-10	1-07-8	1556-8	6.474	0-08-8	4-216-8	3.680-8
3.123	6.1	1.4	7.5	2.840	4.8-11	7-10	5-19-10	6.424	9-10-10	1-07-8	1595-8	6.474	0-08-8	4-221-8	3.680-8
3.179	7.1	1.4	8.5	2.840	4.8-11	7-10	5-19-10	6.474	9-10-10	1-07-8	1634-8	6.474	0-08-8	4-226-8	3.680-8
3.235	8.1	1.4	9.5	2.840	4.8-11	7-10	5-19-10	6.524	9-10-10	1-07-8	1673-8	6.474	0-08-8	4-231-8	3.680-8
3.291	9.1	1.4	10.5	2.840	4.8-11	7-10	5-19-10	6.574	9-10-10	1-07-8	1712-8	6.474	0-08-8	4-236-8	3.680-8
3.347	1.2	1.4	2.2	2.840	4.8-11	7-10	5-19-10	6.624	9-10-10	1-07-8	1751-8	6.474	0-08-8	4-241-8	3.680-8
3.393	2.1	1.4	3.5	2.840	4.8-11	7-10	5-19-10	6.674	9-10-10	1-07-8	1790-8	6.474	0-08-8	4-246-8	3.680-8
3.449	3.1	1.4	4.5	2.840	4.8-11	7-10	5-19-10	6.724	9-10-10	1-07-8	1829-8	6.474	0-08-8	4-251-8	3.680-8
3.505	4.1	1.4	5.5	2.840	4.8-11	7-10	5-19-10	6.774	9-10-10	1-07-8	1868-8	6.474	0-08-8	4-256-8	3.680-8
3.561	5.1	1.4	6.5	2.840	4.8-11	7-10	5-19-10	6.824	9-10-10	1-07-8	1907-8	6.474	0-08-8	4-261-8	3.680-8
3.617	6.1	1.4	7.5	2.840	4.8-11	7-10	5-19-10	6.874	9-10-10	1-07-8	1946-8	6.474	0-08-8	4-266-8	3.680-8
3.673	7.1	1.4	8.5	2.840	4.8-11	7-10	5-19-10	6.924	9-10-10	1-07-8	1985-8	6.474	0-08-8	4-271-8	3.680-8
3.729	8.1	1.4	9.5	2.840	4.8-11	7-10	5-19-10	6.974	9-10-10	1-07-8	2024-8	6.474	0-08-8	4-276-8	3.680-8
3.785	9.1	1.4	10.5	2.840	4.8-11	7-10	5-19-10	7.024	9-10-10	1-07-8	2063-8	6.474	0-08-8	4-281-8	3.680-8
3.841	1.2	1.4	2.2	2.840	4.8-11	7-10	5-19-10	7.074	9-10-10	1-07-8	2102-8	6.474	0-08-8	4-286-8	3.680-8
3.897	2.1	1.4	3.5	2.840	4.8-11	7-10	5-19-10	7.124	9-10-10	1-07-8	2141-8	6.474	0-08-8	4-291-8	3.680-8
3.953	3.1	1.4	4.5	2.840	4.8-11	7-10	5-19-10	7.174	9-10-10	1-07-8	2180-8	6.474	0-08-8	4-296-8	3.680-8
4.009	4.1	1.4	5.5	2.840	4.8-11	7-10	5-19-10	7.224	9-10-10	1-07-8	2219-8	6.474	0-08-8	4-301-8	3.680-8
4.065	5.1	1.4	6.5	2.840	4.8-11	7-10	5-19-10	7.274	9-10-10	1-07-8	2258-8	6.474	0-08-8	4-306-8	3.680-8
4.121	6.1	1.4	7.5	2.840	4.8-11	7-10	5-19-10	7.324	9-10-10	1-07-8	2297-8	6.474	0-08-8	4-311-8	3.680-8
4.177	7.1	1.4	8.5	2.840	4.8-11	7-10	5-19-10	7.374	9-10-10	1-07-8	2336-8	6.474	0-08-8	4-316-8	3.680-8
4.233	8.1	1.4	9.5	2.840	4.8-11	7-10	5-19-10	7.424	9-10-10	1-07-8	2375-8	6.474	0-08-8	4-321-8	3.680-8
4.289	9.1	1.4	10.5	2.840	4.8-11	7-10	5-19-10	7.474	9-10-10	1-07-8	2414-8	6.474	0-08-8	4-326-8	3.680-8
4.345	1.2	1.4	2.2	2.840	4.8-11	7-10	5-19-10	7.524	9-10-10	1-07-8	2453-8	6.474	0-08-8	4-331-8	3.680-8
4.399	2.1	1.4	3.5	2.840	4.8-11	7-10	5-19-10	7.574	9-10-10	1-07-8	2492-8	6.474	0-08-8	4-336-8	3.680-8
4.455	3.1	1.4	4.5	2.840	4.8-11	7-10	5-19-10	7.624	9-10-10	1-07-8	2531-8	6.474	0-08-8	4-341-8	3.680-8
4.511	4.1	1.4	5.5	2.840	4.8-11	7-10	5-19-10	7.674	9-10-10	1					

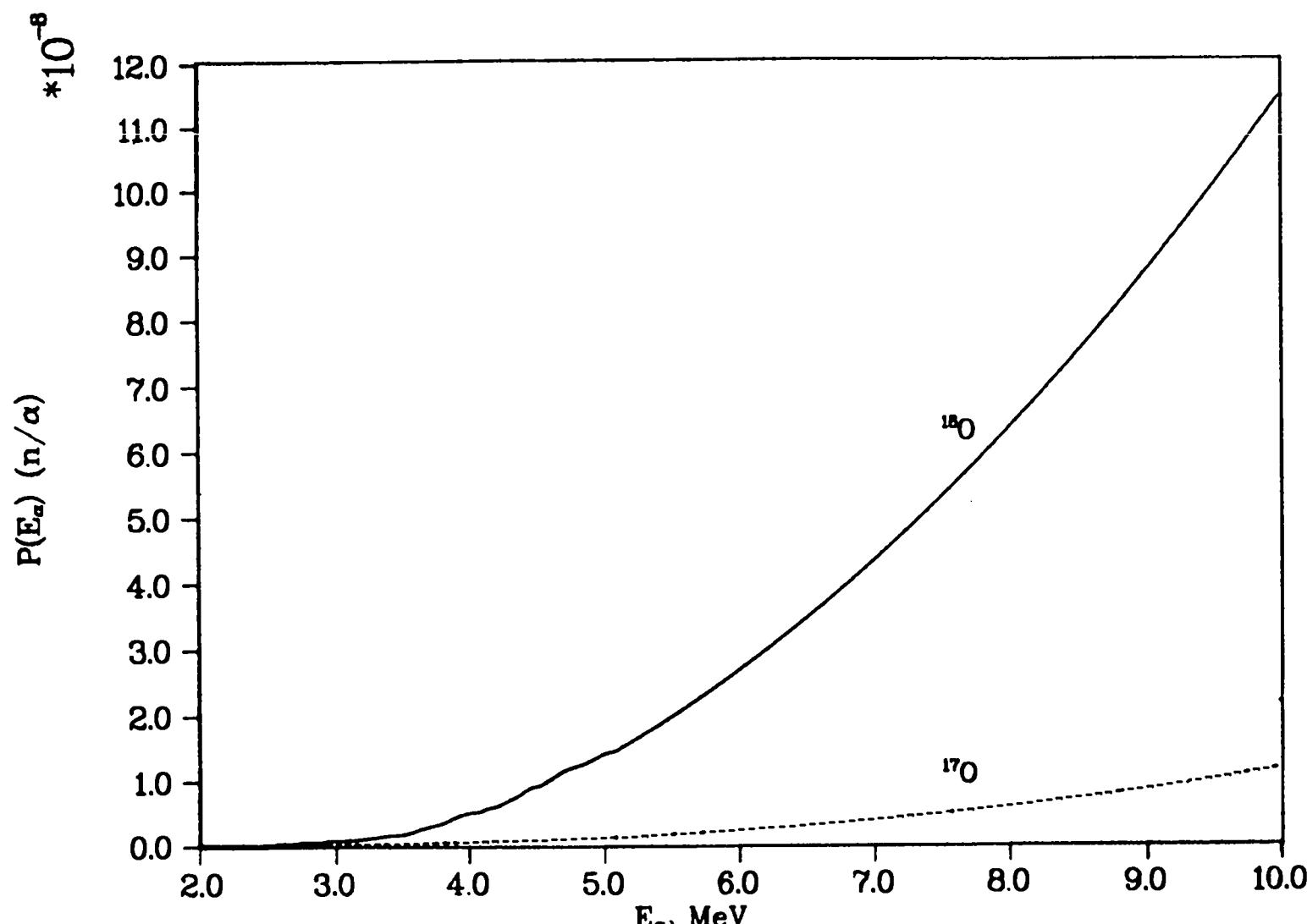


Fig. 5.
 $^{17,18}\text{O}(\alpha,n)$ neutron production by decay alphas in LWR irradiated UO_2 fuel.

TABLE XIII
NEUTRON PRODUCTION FROM ACTINIDE DECAY IN UO₂ FUEL

NUCLIDE	HALF-LIFE (SECONDS)	DECAY ENERGY (MEV)	DE-	◆◆◆NEUTRONS PER DECAY◆◆◆			
				REF	IN UO ₂	FISSION	TOTAL
80-HG-206	4.89000+2	0.5274	A	0.	0.	0.	0.
81-TL-206	2.50980+2	0.5402	A	0.	0.	0.	0.
82-PB-206	STABLE	0.	-	0.	0.	0.	0.
81-TL-207	2.87400+2	0.5194	A	0.	0.	0.	0.
82-PB-207	STABLE	0.	-	0.	0.	0.	0.
81-TL-208	1.84200+2	3.9702	B	0.	0.	0.	0.
82-PB-208	STABLE	0.	-	0.	0.	0.	0.
81-TL-209	1.32000+2	2.8315	A	0.	0.	0.	0.
82-PB-209	1.17108+4	0.2234	A	0.	0.	0.	0.
83-BI-209	6.3115+25	0.	A	0.	0.	0.	0.
81-TL-210	7.80000+1	4.2765	A	0.	0.	7.00 -05	7.00 -05
82-PB-210	7.02472+8	0.0441	A	5.68 -17	0.	5.68 -17	5.68 -17
83-BI-210	4.33123+5	0.3899	A	1.56 -14	0.	1.16 -14	1.16 -14
84-FO-210	1.19557+7	5.4076	A	1.87 -08	0.	1.87 -08	1.87 -08
82-PB-211	2.16600+3	0.5353	A	0.	0.	0.	0.
83-BI-211	1.29000+2	6.7881	A	3.88 -08	0.	3.88 -08	3.88 -08
84-FO-211	0.5160000	7.5942	A	5.64 -08	0.	5.64 -08	5.64 -08
82-PB-212	3.83040+4	0.3180	B	0.	0.	0.	0.
83-BI-212	3.63600+3	2.9030	A	1.076 -08	0.	1.076 -08	1.076 -08
84-FO-212	2.96000-7	8.9536	A	8.94 -08	0.	8.94 -08	8.94 -08
83-BI-213	2.73540+3	0.7172	A	5.85 -10	0.	5.85 -10	5.85 -10
84-FO-213	4.20000-6	8.5360	A	7.86 -08	0.	7.86 -08	7.86 -08
82-PB-214	1.60800+3	0.5389	A	0.	0.	0.	0.
83-BI-214	1.18200+3	2.1923	A	4.39 -12	0.	4.39 -12	4.39 -12
84-FO-214	1.63700-4	7.8337	A	6.19 -08	0.	6.19 -08	6.19 -08
83-BI-215	4.44000+2	0.8445	A	0.	0.	0.	0.
84-FO-215	1.77600-3	7.5265	A	5.52 -08	0.	5.52 -08	5.52 -08
85-AT-215	1.00000-4	8.1780	A	6.98 -08	0.	6.98 -08	6.98 -08
84-FO-216	0.1500000	6.9064	B	4.28 -08	0.	4.28 -08	4.28 -08
85-AT-217	0.0323000	7.2004	A	4.85 -08	0.	4.85 -08	4.85 -08
86-RN-217	5.40000-4	7.8880	A	6.32 -08	0.	6.32 -08	6.32 -08
84-FO-218	1.83000+2	6.1149	A	2.909 -08	0.	2.909 -08	2.909 -08
85-AT-218	1.7500000	6.8830	A	4.14 -08	0.	4.14 -08	4.14 -08
86-RN-218	0.0350000	7.2664	A	4.99 -08	0.	4.99 -08	4.99 -08
85-AT-219	5.40000+1	6.2165	A	3.26 -08	0.	3.26 -08	3.26 -08
86-RN-219	3.9600000	6.9463	A	4.25 -08	0.	4.25 -08	4.25 -08
86-RN-220	5.56000+1	6.4048	B	3.39 -08	0.	3.39 -08	3.39 -08
87-FR-221	2.88000+2	6.4580	A	3.45 -08	0.	3.45 -08	3.45 -08
86-RN-222	3.30351+5	5.5905	A	2.129 -08	0.	2.129 -08	2.129 -08
87-FR-222	8.64000+2	0.7450	A	2.45 -11	0.	2.45 -11	2.45 -11
88-RA-222	3.80000+1	6.6760	A	3.846 -08	0.	3.846 -08	3.846 -08
87-FR-223	1.30800+3	0.4559	A	7.65 -13	0.	7.65 -13	7.65 -13
88-RA-223	9.87949+5	-----	A	2.39 -08	0.	2.39 -08	2.39 -08
88-RA-224	3.16224+5	5.7903	B	2.40 -08	0.	2.40 -08	2.40 -08
88-RA-225	1.27872+6	0.1433	A	0.	0.	0.	0.
89-AC-225	8.64000+5	5.9354	A	2.57 -08	0.	2.57 -08	2.57 -08
88-RA-226	5.0461+10	4.8708	A	1.304 -08	0.	1.304 -08	1.304 -08
89-AC-226	1.04400+5	0.4099	A	1.24 -12	0.	1.24 -12	1.24 -12

TABLE XIII (cont.)

NUCLIDE	HALF-LIFE (SECONDS)	DECAY ENERGY (MEV)	REF	DE- CAY CRY ALPHA, N SPONT.				♦♦♦♦♦NEUTRONS PER DECAY♦♦♦♦♦	
				IN UO2	FISSION	TOTAL			
90-TH-226	1.85400+3	6.4517	A	3.42 -08	0.	3.42	-08		
89-AC-227	6.87097+8	0.0878	A	2.01 -10	0.	2.01	-10		
90-TH-227	1.61720+6	6.1466	A	2.72 -08	0.	2.72	-08		
88-PA-228	1.82087+8	0.0146	A	0.	0.	0.			
89-AC-228	2.20680+4	1.3696	A	0.	0.	0.			
90-TH-228	6.03725+7	5.5176	B	2.004-08	0.	2.004	-08		
90-TH-229	2.3163+11	5.1686	A	1.391-08	0.	1.391	-08		
90-TH-230	2.4299+12	4.7609	B	1.207-08	1.14 -12	1.21	-08		
91-PA-230	1.52928+6	0.6577	A	6.03 -13	0.	6.03	-13		
92- U-230	1.79712+6	5.9928	A	2.69 -08	0.	2.69	-08		
90-TH-231	9.18720+4	0.1537	B	0.	0.	0.			
91-PA-231	1.0338+12	5.0601	B	1.478-08	5.75 -12	1.48	-08		
92- U-231	3.62880+5	0.1017	A	1.14 -12	0.	1.14	-12		
90-TH-232	4.4337+17	4.0882	B	5.52 -09	3.02 -11	5.55	-09		
91-PA-232	1.13184+5	1.098	B	0.	0.	0.			
92- U-232	2.26263+9	5.4145	B	1.871-08	1.54 -12	1.87	-08		
90-TH-233	1.33800+3	0.4422	B	0.	0.	0.			
91-PA-233	2.33280+6	0.4080	B	0.	0.	0.			
92- U-233	5.0232+12	4.8978	B	1.336-08	2.29 -12	1.34	-08		
90-TH-234	2.08233+6	0.1473	A	0.	0.	0.			
91-PA-234	2.43000+4	2.2453	A	0.	0.	0.			
91-PA-234M	7.05000+1	0.8141	A	0.	0.	0.			
92- U-234	7.7188+12	4.8685	B	1.299-08	2.17 -11	1.301	-08		
90-TH-235	4.14000+2	-----	A	0.	0.	0.			
91-PA-235	1.45200+3	-----	A	0.	0.	0.			
92- U-235	2.2210+16	4.6651	B	8.89 -09	3.74 -09	1.26	-08		
92- U-235M	1.48080+3	0.0001	A	0.	0.	0.			
93-NP-235	3.42230+7	0.0810	A	2.44 -13	0.	2.44	-13		
94-PU-235	1.53600+3	5.8675	A	3.48 -12	0.	3.48	-12		
92- U-236	7.3890+14	4.5809	B	9.89 -09	2.29 -09	1.218	-08		
93-NP-236	3.6290+12	0.3390	B	0.	0.	0.			
93-NP-236M	8.10000+4	0.1353	B	0.	0.	0.			
94-PU-236	8.99688+7	5.8634	B	2.517-08	1.73 -09	2.69	-08		
92- U-237	5.83200+5	0.3103	B	0.	0.	0.			
93-NP-237	6.7532+13	4.9470	B	1.303-08	4.39 -12	1.303	-08		
94-PU-237	3.94243+6	0.0628	B	6.72 -13	0.	6.72	-13		
92- U-238	1.4100+17	4.2755	B	6.64 -09	1.095-06	1.102	-06		
93-NP-238	1.82908+5	0.7916	B	0.	0.	0.			
94-PU-238	2.76912+9	5.5871	B	2.124-08	4.08 -09	2.532	-08		
92- U-239	1.41000+3	0.4650	B	0.	0.	0.			
93-NP-239	2.03385+5	0.4180	B	0.	0.	0.			
94-PU-239	7.6084+11	5.2396	B	1.664-08	9.37 -12	1.665	-08		
92- U-240	5.07600+4	0.1755	A	0.	0.	0.			
93-NP-240	4.02000+3	1.5755	A	0.	0.	0.			
93-NP-240M	4.50000+2	1.0407	A	0.	0.	0.			
94-PU-240	2.0670+11	5.3274	B	1.676-08	1.08 -07	1.25	-07		
95-AM-240	1.82880+5	1.0920	B	3.74 -14	0.	3.74	-14		
96-CM-240	2.31552+6	6.3844	A	3.37 -08	9.23 -08	1.26	-07		
94-PU-241	4.63886+8	0.0054	B	3.39 -13	0.	3.39	-13		
95-AM-241	1.3639+10	5.6131	B	2.115-08	9.31 -12	2.116	-08		
96-CM-241	2.83392+6	1.1100	B	2.79 -10	0.	2.79	-10		
94-PU-242	1.1875+13	4.9812	B	1.406-08	1.18 -05	1.18	-05		
95-AM-242	5.76360+4	0.1944	B	0.	0.	0.			
95-AM-242M	4.79665+9	0.0631	B	9.22 -11	3.74 -10	4.56	-10		

TABLE XIII (cont.)

NUCLIDE	HALF-LIFE (SECONDS)	ENERGY (MEV)	REF	DECAY DATA REFERENCES					
				CAY	ALPHA,N	SPONT.	IN UO2	FISSION	TOTAL
96-CM-242	1.40745+7	6.2169	B	3.07 -08	1.714-07	2.02 -07			
94-PU-243	1.78452+4	0.1957	B	0.	0.	0.			
95-AM-243	2.3289+11	5.4224	B	1.82 -08	5.32 -10	1.87 -08			
96-CM-243	8.99372+8	6.1598	B	2.62 -08	0.	2.62 -08			
94-PU-244	2.5877+15	4.6510	B	1.063-08	2.875-03	2.88 -03			
95-AM-244	3.63600+4	1.1177	B	0.	0.	0.			
95-AM-244M	1.56000+3	0.5088	B	0.	0.	0.			
96-CM-244	5.71495+8	5.9010	B	2.582-08	3.623-06	3.65 -06			
94-PU-245	3.78280+4	0.8103	A	0.	0.	0.			
95-AM-245	7.38000+3	0.3199	A	0.	0.	0.			
96-CM-245	2.6744+11	5.5881	B	1.948-08	0.	1.95 -08			
94-PU-246	9.37440+5	0.2514	A	0.	0.	0.			
95-AM-246M	1.50000+3	1.4433	A	0.	0.	0.			
96-CM-246	1.4926+11	5.4714	B	1.971-08	8.313-04	8.31 -04			
96-CM-247	4.9229+14	5.3522	B	1.466-08	0.	1.47 -08			
96-CM-248	1.0720+13	4.7270	B	1.441-08	2.569-01	2.57 -01			
97-BK-248	2.84018+8	-----	A	-----	-----	-----			
97-BK-248M	8.46000+4	0.1684	A	0.	0.	0.			
98-CF-248	2.88144+7	6.3613	A	3.336-08	9.519-05	9.52 -05			
96-CM-249	3.84900+3	0.2932	B	0.	0.	0.			
97-BK-249	2.76480+7	0.0331	B	2.906-13	1.656-09	1.66 -09			
98-CF-249	1.1064+10	6.2903	B	2.646-08	1.712-08	4.36 -08			
96-CM-250	3.5660+11	-----	C	-----	2.32 +00	2.32 +00			
97-BK-250	1.15812+4	1.1829	B	0.	0.	0.			
98-CF-250	4.12764+8	6.1227	B	2.941-08	2.718-03	2.72 -03			
96-CM-251	1.00800+3	0.5925	A	0.	0.	0.			
97-BK-251	3.33600+3	0.4988	A	0.	0.	0.			
98-CF-251	2.8338+10	6.0260	B	2.532-08	0.	2.53 -08			
98-CF-252	8.32471+7	6.0317	B	2.996-08	1.164-01	1.164-01			
98-CF-253	1.53878+6	0.0980	B	8.89 -11	0.	8.89 -11			
99-ES-253	1.76860+6	6.7367	B	3.995-08	3.419-07	3.82 -07			
98-CF-254	5.22720+6	0.0184	A	8.167-11	3.88 +00	3.88 +00			
99-ES-254	2.38205+7	6.6172	A	3.627-08	1.193-07	1.56 -07			
99-ES-254M	1.41480+5	0.7351	A	1.138-10	1.778-07	1.78 -07			
100-FM-254	1.16640+4	7.2996	A	5.08 -08	2.34 -03	2.34 -03			
98-CF-255	6.84000+3	-----	A	0.	0.	0.			
99-ES-255	3.30912+6	0.5956	A	2.72 -09	1.59 -04	1.59 -04			
100-FM-255	7.22520+4	7.2407	A	4.75 -08	8.54 -07	9.02 -07			
99-ES-256	1.32000+3	0.6169	A	0.	0.	0.			
100-FM-256	9.45720+3	7.0250	A	4.55 -08	3.69 +00	3.69 +00			
100-FM-257	8.68320+6	6.8640	A	3.81 -08	8.09 -03	8.09 -03			
100-FM-258	3.80000-4	-----	A	0.	4.03 +00	4.03 +00			

DECAY DATA REFERENCES

A=TABLE OF ISOTOPES

B=ENDF/B-V

C=A.TOBIAS,U.K.,PRIVATE COMMUNICATION

ADDITIONAL NOTES

MISSING DATA NOTED AS -----

81-TL-210, NEUTRONS FROM DELAYED NEUTRON
EMISSION FROM 82-PB-210 LEVELS
PRODUCED IN BETA DECAY.92- U-235, SPONTANEOUS FISSION BRANCHING
IN ENDF/B-V IS ZERO BY OMISSION.
S.F. BRANCHING(2.011-9) TAKEN
FROM REFERENCE A.

97-BK-248 DECAY CHARACTERISTICS UNKNOWN.

These values of R_k may be used with detailed calculated activity inventory to determine total neutron production within oxide fuel, using Eq. (22).

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