Los Alamos National Labbratory is operated by the University of California for the United States Department of Energy under contract W-F405-ENG-36

TITLE NEW DEVELOPMENTS ENHANCING MCNP FOR CRITICALITY SAFETY

AUTHOP(S)

J. S. Hendricks

G. W. McKinney

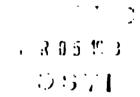
R. Acthur Forster

SUBMITTED TO

1993 Topical Meeting of the American Nuclear Society, Nashville, TN, September 19-23, 1993

DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, nor any of their employees, makes any warranty, express or implied, or assumes any regal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise these not necessarily constitute or imply its endorsement, recommendation, or lawring by the United States Government or any agency thereof. The views and ordinants of authors expressed bettern the tool necessarily state or reflect those of the United States Government or any agency thereof.



By acceptance of this artifulate intermediate recognizer that the U.S. Sovernment retains a numericularity toyally-free ocense to publish or reproduce the process form of the control some or the army offers to the U.S. Revenuent proposes.

the line Author National Information reminests that the probation intently this entrining with porturnal mider the guap resignifical Side growers of Charge

MASTER LOS Alamos National Laboratory Los Alamos, New Mexico 87545

11 MI JAZE CIE

DISTRIBUTION OF THIS DUCUMENT IS UNLIMITED

NEW DEVELOPMENTS ENHANCING MCNP FOR CRITICALITY SAFETY

by

John S. Hendricks, Gregg W. McKinney, and R. Arthur Forster Los Alamos National Laboratory Los Alamos, NM 87545

I. INTRODUCTION

A number of significant new developments have been made to enhance the MCNP Monte Carlo radiation transport code¹ for criticality safety applications. These are available in the newly released MCNP4A version of the code.

II. SPECIFIC CRITICALITY IMPROVEMENTS

A number of MCNP projects have been undertuken at the request of and specifically for the criticality safety community.

A. New Criticality Analysis and Output

Since the early 80's MCNP has had three estimates of k_{eff} : collision, absorption, and track length. MCNP has also had collision and absorption estimators of removal lifetime. These are calculated for every cycle and are averaged over the cycles as snaple averages and covariance weighted averages. Correlation coefficients between estimators are also calculated. These criticality estimators are all in addition to the extensive summary information and tally edits used in shielding and other problems.

In MCNP4A the following have been added:

- A summary page of k_{eff} results with confidence intervals. The summary includes:
 a check to determine if each cell with fissionable material had tracks entering.
 collisions, and fission source points to assess problem sampling;
 - tests for normality of the active k_{eff} values for each estimator;
 - a table of k_{eff} and confidence intervals if the largest value of k_{eff} for each estimator occurs on the next cycle;
 - a fission neutron lifetime estimate
- A table of k_{eff} and its variance as it would have been calculated with a different number of k_{eff} sources per batch to assess k_{eff} cycle correlation effects;
- A table showing k_{eff} results by cycle number as well as the largest and smallest values
 of each k_{eff} estimator and the figure of merit (a measure of problem efficiency);
- A printed plot of the cumulative average k_{eff} by cycle;
- A table of k_{eff} as a function of the number of (inactive) cycles skipped along with
 the number of inactive cycles that produces the minimum k_{eff} variance. This table
 also includes an examination of the first two moments of k_{eff} for the first and second
 halves of the active cycles to assess settling to the spatial distribution normal mode;
- A printed plot of the combined k_{iff} as a function of the number of (inactive) cycles skipped.

In addition, there are new messages sent to the user's terminal, additional error diagnostics and warning messages about criticality problem performance. Most of the new information can be interactively plotted by the MCNP tally plotting package either as a run progresses or afterward.

B. Benchmark Calculations

A major benchmarking program has been undertaken for MCNP. A comparison² of MCNP to KENO using the KENO test problem set is presented elsewhere at this meeting.³ Additional criticality benchmarks can be found in the MCNP neutron benchmarks.^{4,5} Collaborators at General Electric have published light water reactor critical benchmarks⁶ and collaborators at Westinghouse Idaho Nuclear Company have published several volumes^{7,8,9,10} comparing MCNP to SCALE and experiments. These benchmarks all demonstrate that MCNP is an accurate computer model for calculating nuclear criticality.

C. Criticality Primer

A primer on how to use MCNP for criticality safety problems is being prepared in collaboration with Bob Busch of the University of New Mexico. The primer will give new users a step by-step guide for setting up and interpreting MCNP problems starting with simple examples (critical spheres) and building up to more sophisticated problems likely to be encountered by criticality practitioners.

III. GENERAL MONP IMPROVEMENTS

A number of MCNP enhancements are of interest to all applications of MCNP, including criticality safety.

A. A New Means of Assessing Tally Quality

MCNP4A features a new, exclusive error analysis in addition to the usual estimate of tally variance. The variance of the variance and the underlying history score probability density function are calculated and analyzed as new pids to assess tally convergence.¹¹

Each tally is also subjected to ten statistical checks providing new confidence and insight into the statistical convergence process.

B. X-Windows Graphics

MCNP geometry and tally plotting now work with Xwindow-based window systems (e.g., Open Windows, Motif, etc.) in addition to GKS, CGS, DISSPLA, and PLOT10. The X libraries are well-established and can be found on most workstations and mainframes. This capability has been tested on SUNs, IBM RS/6000s, HP, and Cray UNICOS. The graphics window can be easily resized, iconized, or printed and can be sent to or from other displays.

C. PVM Multiprocessing

MCNP can now be run in parallel on a cluster of workstutions¹² using PVM (Parallel Virtual Machine) software. On the 16-machine IBM cluster at LANL, MCNP runs ten times as fast as a single-processor YMP. PVM multiprocessing has also been demonstrated on a network of SUN workstations.

D. Dynamic Memory on Unix Systems

MCNP now dynamically adjusts its memory to problem size on UNIX systems. The new UNIX dynamic memory manager also works in conjunction with the distributed memory multiprocessing.

E. ENDF/B-VI Physics

The Kulbach 87 formulism (ENDF/B VI file 6 LAW) 1, LANG 2) and correlated energy angle scattering (ENDF/B VI file 6 LAW) 7) have been been been developed in MCNP to leadle new ENDF/B VI data formats. A new algorithm has been developed for next

event estimators so that collisions using these laws now contribute to point and ring detectors as well as the DXTRAN variance reduction method. Preliminary libraries are available and work is proceeding to process, test, and release entirely new ENDF/B-VI libraries.

F. Repeated Structure Tallies

MCNP tallies can now be made in individual lattice elements or repeated cells or combinations thereof in repeated structures geometries.

G. SABRINA Particle Tracks

History files are now optionally output from MCNP to describe particle trajectories which can be plotted with the three-dimensional color graphics code SABRINA. This new visualization capability will provide important insight into Monte Carlo solutions.

H. White and Periodic Boundaries

MCNP now models both white and periodic boundaries. White boundaries are useful for comparison with deterministic methods. Periodic boundaries are valuable in a wide range of reactor lattice calculations.

L Other Features

A completely new MCNP manual is nearing completion. New Software Quality Assumence (SQUA) standards are being applied, along with even more rigorous quality control, configuration management, and installation verification. For example, every change in MCNP for the past three years can be traced to written documentation, and the code is subject to manera is and periodic reviews.

New installation procedures have been developed which we hope are even easier than those for MCNP4.

Protection for long runs with workstations, including better random number control¹³ and cularged output formuts, is now available in MCNP4A. Restart capabilities are improved.

And for the first time MCNP is case insensitive, which is particularly of interest on VMS systems.

IV. SUMMARY

The MCNP Monte Carlo code is increasingly being used by the criticality safety community. Every three years or so a new version is released internationally. The MCNP4A version released in summer 1993 has many enhancements specifically for criticality safety. These are in addition to mancrous other enhancements which benefit all MCNP applications including criticality safety. Not described here are the many additional enhancements, such as new electron transport options and extended photon energy physics, which do not apply to criticality safety. All of these capabilities are available now in MCNP4A. And as active development of MCNP continues, along with a vigorous research program in such areas as nonlinear and non-Boltzman Monte Carlo, hybrid Monte Carlo S_n methods, angle bias, and quasi deterministic variance reduction, more enhancements will be available in future versions of the code.

REFERENCES

 J. F. Briesmeister, Editor, "MCNP A General Purpose Monte Carlo Code for Neutron and Photon Transport," LA 7396 M, Rev. 2 (September 1986)

- John C. Wagner, James E. Sisolak, and Gregg W. McKinney, "MCNP: Criticality Safety Benchmark Problems," LA-12415 (October 1992)
- Gregg W. McKinney, James E. Sisolak, and John C. Wagner, "MCNP/ KENO Criticality Benchmarks," submitted to the ANS 1993 Topical Meeting on Physics 1 and Methods in Criticality Safety, Nashville, Tennessee (September 19 - 23, 1993)
- 4. D. J. Whalen, D. A. Cardon, J. L. Uhle, J. S. Hendricks, "MCNP: Neutron Benchmark Problems," Los Alamos National Laboratory report, LA-12212 (November 1991)
- John S. Hendricks, Daniel J. Whalen, David A. Cardon, Jennifer L. Uhle, "MCNP Neutron Benchmarks," Trans. Am. Nuc. Soc., 65, 262 (1992)
- Shiva Sitaraman, "MCNP: Light Water Reactor Critical Benchmarks," NEDO-32028,
 General Electric Nuclear Energy (March 1992)
- 7. Catherine Crawford and Brian Palmer, "Validation of MCNP, a Comparison with SCALE, Part 1: Highly Enriched Uranium Solutions," CSS-92-012 (October 1992)
- Catherine Crawford and Brian Palmer, "Validation of MCNP, a Comparison with SCALE, Part 2: Highly Enriched Uranium Metal Systems," CSS-92 013 (October 1992)
- Catherine Crawford and Brian Pulmer, "Validation of MCNP, a Comparison with SCALE, Part 3: Highly Enriched Uranian Oxide Systems," CSS 92-014 (October 1992)
- Cutherine Crawford and Briun Pulmer, "Validation of MCNP; SPERT D and BORAX V Fuel," CSS 92-045 (November 1992)

- 11. R. A. Forster, S. P. Pederson, and T. E. Booth, "Two Proposed Convergence Criteria for Monte Carlo Solutions," Trans. Am. Nucl. Soc., 66, 277 (1992)
- Gregg W. McKinney and James T. West, "Multiprocessing MCNP on an IBM RS/6000 Cluster," submitted to the ANS 1993 Annual Meeting, San Diego, California (June 20 24, 1993)
- 13. J. S. Hendricks, "Effects of Changing the Random Number Stride in Monte Carlo Calculations," Nuclear Science and Engineering, 109 (1) 86 91 (September 1991)